

technical information; financial data, such as salaries; and personal information concerning individuals associated with the proposals. These matters are exempt under 5 U.S.C. 552b(c), (4) and (6) of the Government in the Sunshine Act.

Dated: May 20, 1996.

M. Rebecca Winkler,

Committee Management Officer.

[FR Doc. 96-13007 Filed 5-22-96; 8:45 am]

BILLING CODE 7555-01-M

Special Emphasis Panel in Materials Sciences; Notice of Meetings

In accordance with the Federal Advisory Committee Act (Pub. L. 92-463, as amended), the National Science Foundation announces that the Special Emphasis Panel in Materials Research will be holding panel meetings for the purpose of reviewing proposals submitted to the Office of Multidisciplinary Activities in the area of Optical Science and Engineering. In order to review the large volume of proposals, panel meetings will be held on June 12-14, 1996 (2). All meetings will be closed to the public and will be held at the National Science Foundation, 4201 Wilson Boulevard, Arlington, Virginia, from 8:00 a.m. to 5:00 p.m. each day.

Contact Person: Dr. John Weiner, Head, Office of Multidisciplinary Activities, Office of the Assistant Director for Mathematical and Physical Sciences, National Science Foundation, Room 1005, 4201 Wilson Boulevard, Arlington, VA 22230, (703) 306-1800.

Reason for Closing: The proposals being reviewed include information of a proprietary or confidential nature, including technical information, financial data such as salaries, and personal information concerning individuals associated with the proposals. These matters are exempt under 5 U.S.C. 552b(c) (4) and (6) of the Government in the Sunshine Act.

Dated: May 20, 1996.

M. Rebecca Winkler,

Committee Management Officer.

[FR Doc. 96-13006 Filed 5-22-96; 8:45 am]

BILLING CODE 7555-01-M

Special Emphasis Panel in Science and Technology Infrastructure; Notice of Meeting

In accordance with the Federal Advisory Committee Act (Pub. L. 92-463, as amended), the National Science Foundation announces the following meeting:

Name: Special Emphasis Panel in Science and Technology Infrastructure (#1373).

Date and Time: June 9, 1996; 7:30 p.m.-10:00 p.m.; June 10-12, 1996, 8:30 a.m.-5:00 p.m.

Place: Rooms 375 & 380, National Science Foundation, 4201 Wilson Blvd., Arlington, VA 22230; Gallery 1, Renaissance Hotel, 950 North Stafford Street, Arlington, VA 22203.

Type of Meeting: Closed.

Contact Person: Dr. Nathaniel G. Pitts, Director, Office of Science and Technology Infrastructure, Room 1270, 4201 Wilson Blvd., Arlington, VA 22230; Telephone (703) 306-1040.

Purpose of Meeting: To provide advice and recommendations concerning proposals submitted to NSF for financial support.

Agenda: To review and evaluate proposals submitted to the Facilities component of the Academic Research Infrastructure Program.

Reason for Closing: The meeting is closed to the public because the Panel is reviewing proposal actions that will include privileged intellectual property and personal information that could harm individuals if they were disclosed. These matters are exempt under 5 U.S.C. 552B(c) (4) and (6) of the Government in the Sunshine Act.

Dated: May 20, 1996.

M. Rebecca Winkler,

Committee Management Officer.

[FR Doc. 96-13002 Filed 5-22-96; 8:45 am]

BILLING CODE 7555-01-M

Advisory Committee for Social, Behavioral & Economic Sciences; Notice of Meeting

In accordance with the Federal Advisory Committee Act (Pub. L. 92-463, as amended), the National Science Foundation announces the following meeting.

Name and Committee Code: Advisory Committee for Social, Behavioral & Economic Sciences (#1171).

Date and Time: June 6-7, 1996; 900 a.m. to 5:00 p.m.

Place: Room 920 & 310, National Science Foundation, 4201 Wilson Boulevard, Arlington, VA 22230.

Type of Meeting: Closed.

Contact Person: Dr. Steven J. Breckler, National Science Foundation, 4201 Wilson Boulevard, Arlington, VA 22230, Telephone: (703) 306-1731.

Purpose of Meeting: To carry out Committee of Visitors (COV) review, including examination of decisions on proposals, reviewer comments, and other privileged materials.

Agenda: To provide oversight review of the Social Psychology Program.

Reason for Closing: The meeting is closed to the public because the Committee is reviewing proposal actions that will include privileged intellectual property and personal information that could harm individuals if they are disclosed. If discussions were open to the public, these matters that are exempt under 5 U.S.C. 552b(c), (4) and (6) of the Government in the Sunshine Act would be improperly disclosed.

Reason for Late Notice: Difficulty in arranging for a suitable meeting time for the full committee.

Dated: May 20, 1996.

M. Rebecca Winkler,

Committee Management Officer.

[FR Doc. 96-13001 Filed 5-22-96; 8:45 am]

BILLING CODE 7555-01-M

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-295 and 50-304]

Commonwealth Edison Company; Zion Nuclear Power Station, Units 1 and 2 Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an exemption from certain requirements of its regulations to Facility Operating License Nos. DPR-39 and DPR-48, issued to Commonwealth Edison Company (ComEd, the licensee), for operation of Zion Nuclear Power Station, Units 1 and 2, located in Lake County, Illinois.

Environmental Assessment

Identification of the Proposed Action

The proposed action would allow the licensee to utilize American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code) Case N-514, "Low Temperature Overpressure Protection" to determine its low temperature overpressure protection (LTOP) setpoints and is in accordance with the licensee's application for exemption dated March 19, 1996. The proposed action requests an exemption from certain requirements of 10 CFR 50.60, "Acceptance Criteria for Fracture Prevention Measures for Lightwater Nuclear Power Reactors for Normal Operation," to allow application of an alternate methodology to determine the LTOP setpoints for Zion Nuclear Power Station, Units 1 and 2. The proposed alternate methodology is consistent with guidelines developed by the ASME Working Group on Operating Plant Criteria (WGOPC) to define pressure limits during LTOP events that avoid certain unnecessary operational restrictions, provide adequate margins against failure of the reactor pressure vessel, and reduce the potential for unnecessary activation of pressure relieving devices used for LTOP. These guidelines have been incorporated into Code Case N-514, "Low Temperature Overpressure Protection," which has been approved by the ASME Code Committee. The content of this Code Case has been incorporated into

Appendix G of Section XI of the ASME Code and published in the 1993 Addenda to Section XI. However, 10 CFR 50.55a, "Codes and Standards," and Regulatory Guide 1.147, "Inservice Inspection Code Case Acceptability" have not been updated to reflect the acceptability of Code Case N-514.

The philosophy used to develop Code Case N-514 guidelines is to ensure that the LTOP limits are still below the pressure/temperature (P/T) limits for normal operation, but allow the pressure that may occur with activation of pressure relieving devices to exceed the P/T limits, provided acceptable margins are maintained during these events. This philosophy protects the pressure vessel from LTOP events, and still maintains the Technical Specifications P/T limits applicable for normal heatup and cooldown in accordance with 10 CFR Part 50, Appendix G and Sections III and XI of the ASME Code.

The Need for the Proposed Action

Pursuant to 10 CFR 50.60, all lightwater nuclear power reactors must meet the fracture toughness requirements for the reactor coolant pressure boundary as set forth in 10 CFR Part 50, Appendix G. 10 CFR Part 50, Appendix G, defines P/T limits during any condition of normal operation including anticipated operational occurrences and system hydrostatic tests, to which the pressure boundary may be subjected over its service lifetime. It is specified in 10 CFR 50.60(b) that alternatives to the described requirements in 10 CFR Part 50, Appendix G, may be used when an exemption is granted by the Commission under 10 CFR 50.12.

To prevent transients that would produce excursions exceeding the 10 CFR Part 50, Appendix G, P/T limits while the reactor is operating at low temperatures, the licensee installed an LTOP system. The LTOP system includes pressure relieving devices in the form of Power Operated Relief Valves (PORVs) that are set at a pressure below the LTOP enabling temperature that would prevent the pressure in the reactor vessel from exceeding the P/T limits of 10 CFR Part 50, Appendix G. To prevent these valves from lifting as a result of normal operating pressure surges (e.g., reactor coolant pump starting and shifting operating charging pumps) with the reactor coolant system in a solid water condition, the operating pressure must be maintained below the PORV setpoint.

In addition, to prevent damage to reactor coolant pump seals, the operator must maintain a minimum differential

pressure across the reactor coolant pump seals. Hence, the licensee must operate the plant in a pressure window that is defined as the difference between the minimum required pressure to start a reactor coolant pump and the operating margin to prevent lifting of the PORVs due to normal operating pressure surges. The 10 CFR Part 50, Appendix G, safety margin adds instrument uncertainty into the LTOP setpoint. The licensee's current LTOP analysis indicates that using this 10 CFR Part 50, Appendix G, safety margin to determine the PORV setpoint would result in an operating window between the LTOP setpoint and the minimum pressure required for reactor coolant pump seals which is too small to permit continued operation. Operating with these limits could result in the lifting of the PORVs or damage to the reactor coolant pump seals during normal operation. Using Code Case N-514 would allow the licensee to recapture most of the operating margin that is lost by factoring in the instrument uncertainties in the determination of the LTOP setpoint. The net effect of using Code Case N-514 is that the setpoint will not change significantly with the next setpoint analysis. Therefore, the licensee proposed that in determining the PORV setpoint for LTOP events for Zion, the allowable pressure be determined using the safety margins developed in an alternate methodology in lieu of the safety margins required by 10 CFR Part 50, Appendix G. The alternate methodology is consistent with ASME Code Case N-514. The content of this Code Case has been incorporated into Appendix G of Section XI of the ASME Code and published in the 1993 Addenda to Section XI.

An exemption from 10 CFR 50.60 is required to use the alternate methodology for calculating the maximum allowable pressure for LTOP considerations. By application dated March 19, 1996, the licensee requested an exemption from 10 CFR 50.60 to allow it to utilize the alternate methodology of Code Case N-514 to compute its LTOP setpoints.

Environmental Impacts of the Proposed Action

Appendix G of the ASME Code requires that the P/T limits be calculated: (a) using a safety factor of two on the principal membrane (pressure) stresses, (b) assuming a flaw at the surface with a depth of one quarter (1/4) of the vessel wall thickness and a length of six (6) times its depth, and (c) using a conservative fracture toughness curve that is based on the lower bound of static, dynamic, and

crack arrest fracture toughness tests on material similar to the Zion reactor vessel material.

In determining the PORV setpoint for LTOP events, the licensee proposed the use of safety margins based on an alternate methodology consistent with the proposed ASME Code Case N-514 guidelines. ASME Code Case N-514 allows determination of the setpoint for LTOP events such that the maximum pressure in the vessel will not exceed 110% of the P/T limits of the existing ASME Appendix G. This results in a safety factor of 1.8 on the principal membrane stresses. All other factors, including assumed flaw size and fracture toughness, remain the same. Although this methodology would reduce the safety factor on the principal membrane stresses, use of the proposed criteria will provide adequate margins of safety to the reactor vessel during LTOP transients.

The change will not increase the probability or consequences of accidents, no changes are being made in the types of any effluents that may be released offsite, and there is no significant increase in the allowable individual or cumulative occupational radiation exposure. Accordingly, the Commission concludes that there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential nonradiological impacts, the proposed action does involve features located entirely within the restricted area as defined in 10 CFR Part 20. It does not affect nonradiological plant effluents and has no other environmental impact. Accordingly, the Commission concludes that there are no significant nonradiological environmental impacts associated with the proposed action.

Alternatives to the Proposed Action

Since the Commission has concluded there is no measurable environmental impact associated with the proposed action, any alternatives with equal or greater environmental impact need not be evaluated. As an alternative to the proposed action, the staff considered denial of the proposed action. Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

This action does not involve the use of any resources not previously considered in the Final Environmental Statement for the Zion Nuclear Power Station, Units 1 and 2.

Agencies and Persons Consulted

In accordance with its stated policy, on April 22, 1996, the staff consulted with the Illinois State official, Mr. Frank Niziolek; Head, Reactor Safety Section; Division of Engineering; Illinois Department of Nuclear Safety; regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

Based upon the environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated March 19, 1996, which is available for public inspection at the Commission's Public Document Room, 2120 L Street NW., Washington, DC, and at the local public document room located at the Waukegan Public Library, 128 N. County Street, Waukegan, Illinois 60085.

Dated at Rockville, Maryland, this 16th day of May 1996.

For the Nuclear Regulatory Commission.
Clyde Y. Shiraki,

*Project Manager, Project Directorate III-2,
Division of Reactor Projects—III/IV, Office of
Nuclear Reactor Regulation.*

[FR Doc. 96-12947 Filed 5-22-96; 8:45 am]

BILLING CODE 7590-01-P

[Docket Nos. 50-206, 50-361 and 50-362]

**Southern California Edison Company
and San Diego Gas and Electric
Company, San Onofre Nuclear Station;
Notice of Temporary Closing of Local
Public Document Room**

Notice is hereby given that the Main Library, University of California, Irvine, California, which serves as the Nuclear Regulatory Commission (NRC) local public document room (LPDR) for the Southern California Edison Company's and San Diego Gas and Electric Company's San Onofre Nuclear Station, will close on June 15, 1996, for seismic upgrades to the library building. The library is scheduled to reopen in January 1997.

During this period, every effort will be made to meet the informational needs of LPDR patrons. NRC records from mid-1996 forward will be available on microfiche in the Science Library, University of California, Irvine, California. The locations of other LPDRs that maintain records on San Onofre can

be obtained by contacting the NRC LPDR staff. Their toll-free telephone number is (800) 638-8081. Requests for records may also be addressed to the NRC's Public Document Room (PDR), 2120 L Street NW., Lower Level, Washington, DC 20555-0001. The PDR's toll-free telephone number is (800) 397-4209.

Persons interested in using the San Onofre LPDR collection while the Main Library is closed are asked to contact the NRC LPDR staff at their toll-free telephone number listed above.

Questions concerning the NRC's LPDR program or the availability of agency documents in the Irvine area should be addressed to Ms. Jona L. Souder, LPDR Program Manager, Freedom of Information/Local Public Document Room Branch, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, telephone number (800) 638-8081.

Dated at Rockville, Maryland, this 17th day of May, 1995.

For the Nuclear Regulatory Commission.
Carlton C. Kammerer,
*Director, Division of Freedom of Information
and Publications Services, Office of
Administration.*

[FR Doc. 96-12948 Filed 5-22-96; 8:45 am]

BILLING CODE 7590-01-P

**Advisory Committee on Reactor
Safeguards Joint Meeting of the ACRS
Subcommittees on Individual Plant
Examinations and on Probabilistic
Risk Assessment**

The ACRS Subcommittees on Individual Plant Examinations and on Probabilistic Risk Assessment will hold a joint meeting on June 11-12, 1996, Room T-2B3, 11545 Rockville Pike, Rockville, Maryland.

The entire meeting will be open to public attendance.

The agenda for the subject meeting shall be as follows:

*Tuesday, June 11, 1996—8:30 a.m. until
the conclusion of business*

*Wednesday, June 12, 1996—8:30 a.m.
until 12:00 Noon*

The Subcommittees will discuss the Individual Plant Examination (IPE) Insights Report (June 11, 1996), and the staff's research program related to probabilistic risk assessment (June 12, 1996).

The purpose of this meeting is to gather information, analyze relevant issues and facts, and to formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Oral statements may be presented by members of the public with the

concurrence of the Subcommittee Chairman; written statements will be accepted and made available to the Committee. Electronic recordings will be permitted only during those portions of the meeting that are open to the public, and questions may be asked only by members of the Subcommittees, their consultants, and staff. Persons desiring to make oral statements should notify the cognizant ACRS staff engineer named below five days prior to the meeting, if possible, so that appropriate arrangements can be made.

During the initial portion of the meeting, the Subcommittees, along with any of their consultants who may be present, may exchange preliminary views regarding matters to be considered during the balance of the meeting.

The Subcommittees will then hear presentations by and hold discussions with representatives of the NRC staff, its consultants, and other interested persons regarding this review.

Further information regarding topics to be discussed, whether the meeting has been cancelled or rescheduled, the Chairman's ruling on requests for the opportunity to present oral statements, and the time allotted therefor can be obtained by contacting the cognizant ACRS staff engineers, Mr. Michael T. Markley (telephone 301/415-6885) or Mr. Brian Hughes (telephone 301/415-5767) between 7:30 a.m. and 4:15 p.m. (EDT). Persons planning to attend this meeting are urged to contact one of the above named individuals one or two working days prior to the meeting to be advised of any potential changes to the agenda, etc., that may have occurred.

Dated: May 16, 1996.

Sam Duraiswamy,

Chief, Nuclear Reactors Branch.

[FR Doc. 96-12949 Filed 5-22-96; 8:45 am]

BILLING CODE 7590-01-P

**Advisory Committee on Reactor
Safeguards; Joint Meeting of the ACRS
Subcommittees on Probabilistic Risk
Assessment and on Westinghouse
Standard Plant Designs**

The ACRS Subcommittees on Probabilistic Risk Assessment and on Westinghouse Standard Plant Designs will hold a joint meeting on June 5, 1996, Room T-2B3, 11545 Rockville Pike, Rockville, Maryland.

The entire meeting will be open to public attendance with the exception of a portion that may be closed to discuss Westinghouse proprietary information pursuant to 5 U.S.C. 552b(c)(4).

The agenda for the subject meeting shall be as follows: