

Engineering, (703) 306-1361, National Science Foundation, 4201 Wilson Boulevard, Arlington, VA 22230.

Purpose of Meeting: To provide advice and recommendations concerning proposals submitted to the NSF for financial support.

Agenda: To review and evaluate SBIR Phase I proposals concerning Dynamic Systems and Control and Biochemical Engineering and Biotechnology as part of the selection process for awards.

Reason for Closing: The proposals being reviewed include information of a proprietary or confidential nature, including technical information, financial data such as salaries, and personal information concerning individuals associated with the proposals. These matters are exempt under 5 U.S.C. 552b(c) (4) and (6) of the Government in the Sunshine Act.

Dated: July 29, 1996.
M. Rebecca Winkler,
Committee Management Officer.
[FR Doc. 96-19584 Filed 7-31-96; 8:45 am]
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Special Emphasis Panel in Earth Sciences; Notice of Meeting

In accordance with the Federal Advisory Committee Act (Pub. L. 92-463, as amended), the National Science Foundation announces the following meeting.

Name: Special Emphasis Panel in Earth Sciences. (1756)
Date and Time: August 26-August 28, 1996; 8:30 am to 6:00 pm.
Place: Southern California Earthquake Center, California Institute of Technology.
Type of Meeting: Closed.
Contact Person: Dr. James H. Whitcomb, Program Director, Geophysics Program, Division of Earth Sciences, Room 785, National Science Foundation, Arlington, VA 22230, (703) 306-1556.
Purpose of Meeting: To review the renewal proposal, evaluate the Science and Technology Center, and make a recommendation concerning future funding of the Science and Technology Center.
Agenda: To evaluate (a) the research program; (b) educational and outreach activities; and (c) the knowledge transfer activities and the management of the STC. To

make a recommendation on the future funding of the STC.
Reason for Closing: The proposal being reviewed includes information of a proprietary or confidential nature, including technical information, financial data, such as salaries; and personal information concerning individuals associated with the proposals. These matters are within exemptions (4) and (6) of 5 U.S.C. 552(c), the Government in the Sunshine Act.
Dated: July 29, 1996.
M. Rebecca Winkler,
Committee Management Officer.
[FR Doc. 96-19580 Filed 7-31-96; 8:45 am]
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Special Emphasis Panel in Materials Research; Notice of Meeting

In accordance with the Federal Advisory Committee Act (Pub. L. 92-463 as amended), the National Science Foundation announces the following meeting:
Name: Special Emphasis Panel in Materials Research (DMR).

Date	Time	Place
August 21, 1996	7:00 a.m.-9:00 p.m.	South Coast Inn, (Meeting Room), Goleta, CA.
August 22, 1996	8:00 a.m.-5:00 p.m.	University of California, Santa Barbara, CA 93106.

Type of Meeting: Closed.
Contact Person: Dr. LaVerne D. Hess, Program Director, Electronic Materials Program, Division of Materials Research, Room 1065, National Science Foundation, 4201 Wilson Boulevard, Arlington, VA 22230. Telephone (703) 306-1837.

Purpose of Meeting: To provide advice and recommendations concerning support for the Science and Technology Center (STC) for Quantized Electronic Structures (QUEST) at the University of California, Santa Barbara.

Agenda: To evaluate progress at this Science and Technology Center in relation to continuing support.

Reason for Closing: The project being reviewed may include information of a proprietary or confidential nature, including technical information, financial data such as salaries, and personal information concerning individuals associated with the project. These matters are exempt under 5 U.S.C. 552 b.(c) (4) and (6) of the Government in the Sunshine Act.

Dated: July 29, 1996.
M. Rebecca Winkler,
Committee Management Officer.
[FR Doc. 96-19585 Filed 7-31-96; 8:45 am]
BILLING CODE 7555-01-M

NUCLEAR REGULATORY COMMISSION

Notice of Formation of Working Group To Review Incidents Involving Stolen Industrial Radiography Equipment

AGENCY: U.S. Nuclear Regulatory Commission.
ACTION: Notice of formation of working group.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC), with the Organization of Agreement States, is forming a working group to review the issues related to incidents involving stolen radiography equipment.

FOR INFORMATION CONTACT: Jim Myers, Office of State Programs, Mail Stop OWFN-3-D-23, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555. Telephone: 301-415-2328. Internet: JHM@NRC.GOV.

SUPPLEMENTARY INFORMATION: The U.S. Nuclear Regulatory Commission (NRC), with the Organization of Agreement States, is forming a working group to review the issues related to incidents involving stolen radiography equipment. The initiating event involved Larpn of Texas, a Texas licensee. The group's mission is to objectively review events involving

stolen radiographic equipment and provide recommendations for preventing similar events. The working group's first task will be to develop a charter governing the scope of work, type of products, a work schedule, and internal procedures.
WORKING GROUP ORGANIZATION: The working group consists of the following individuals: Jim Myers, NRC, Office of State Programs; Patricia Santiago, NRC, Office of Nuclear Material Safety and Safeguards; Brad Caskey, Texas; Walter Cofer, Florida; and Mike Henry, Louisiana. The group will be Co-Chaired by Jim Myers, NRC, and Brad Caskey, Texas.
WORKING GROUP MEETINGS: The working group will hold meetings and gather comments from regulatory agencies, the radiography industry, and interested members of the public. Maximum use will be made of other media for facilitating interaction between members of the working group, e.g., conference calls, facsimiles, and electronic mail. Working group meetings will be open to the public and will be held in the Washington, D.C. area, or other locations as agreed upon by the working group members. Individuals attending working group meetings are welcome to provide

comments to the working group's consideration orally, or in writing, at times specified by the working group, Co-Chairs. Seating at the working group meetings will be on a first-come, first-served basis.

MEETING ANNOUNCEMENTS: No meeting is scheduled at this time. Announcements for the first, and subsequent, meetings will be made through the NRC's Meeting Announcement system. The meeting announcement system can be reached three ways:

1. Voice: 800-952-9674.
2. Electronic Bulletin Board: 800-952-9676.
3. Electronic Bulletin Board at FedWorld: 800-303-9672.

Dated at Rockville, Maryland, this 23rd day of July, 1996.

For the U.S. Nuclear Regulatory Commission.

Richard L. Bangart,

Director, Office of State Programs.

[FR Doc. 96-19587 Filed 7-31-96; 8:45 am]

BILLING CODE 7590-01-P

Proposed Generic Communication; Primary Water Stress Corrosion Cracking of Control Rod Drive Mechanism and Other Vessel Head Penetrations

AGENCY: Nuclear Regulatory Commission.

ACTION: Notice of opportunity for public comment.

SUMMARY: The Nuclear Regulatory Commission (NRC) is proposing to issue a generic letter concerning primary water stress corrosion cracking in control rod drive mechanisms and other vessel head penetrations of nuclear power reactors. The purpose of the proposed generic letter is to (1) request that addressees describe their program for ensuring the timely inspection of PWR control rod drive mechanism (CRDM) and other vessel head penetrations and (2) require that all addressees provide to the NRC a written response to this generic letter. The NRC is seeking comment from interested parties regarding both the technical and regulatory aspects of the proposed generic letter presented under the Supplementary Information heading.

The proposed generic letter was endorsed by the Committee to Review Generic Requirements (CRGR) on July 25, 1996. The relevant information that was sent to the CRGR will be placed in the NRC Public Document Room. The NRC will consider comments received from interested parties in the final evaluation of the proposed generic letter. The NRC's final evaluation will

include a review of the technical position and, as appropriate, an analysis of the value/impact on licensees.

Should this generic letter be issued by the NRC, it will become available for public inspection in the NRC Public Document Room.

DATES: Comment period expires September 3, 1996. Comments submitted after this date will be considered if it is practical to do so, but assurance of consideration cannot be given except for comments received on or before this date.

ADDRESSES: Submit written comments to Chief, Rules Review and Directives Branch, U.S. Nuclear Regulatory Commission, Mail Stop T-6D-69, Washington, DC 20555-0001. Written comments may also be delivered to 11545 Rockville Pike, Rockville, Maryland, from 7:30 am to 4:15 pm, Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, 2120 L Street, N.W. (Lower Level), Washington, D.C.

FOR FURTHER INFORMATION CONTACT: C. E. (Gene) Carpenter (301) 415-2169.

SUPPLEMENTARY INFORMATION:

Generic Letter 96-##: Primary Water Stress Corrosion Cracking of Control Rod Drive Mechanism and Other Vessel Head Penetrations (TACS No. M95280)

Addressees

All holders of operating licenses for pressurized water reactors (PWRs), except those licenses that have been amended to possession-only status.

Purpose

The U.S. Nuclear Regulatory Commission (NRC) is issuing this generic letter to (1) request addressees to describe their program for ensuring the timely inspection of PWR control rod drive mechanism (CRDM) and other vessel head penetrations and (2) require that all addressees provide to the NRC a written response to this generic letter relating to the requested information.

Background

Most PWRs have Alloy 600 CRDM nozzle and other vessel head penetrations (VHPs) that extend above the reactor pressure vessel head. The stainless steel housing of the CRDM is screwed and seal-welded onto the top of the nozzle penetration, as shown in Figure 1. The weld between the nozzle and the housing is a dissimilar metal weld, which is also called a bimetallic weld. The nozzles protrude below the vessel head, thus exposing the inside surface of the nozzles to reactor coolant.

The control rod drive (CRD) nozzles and other VHPs are basically the same for all PWRs worldwide, which use a U.S. design (except in Germany and Russia).

Generally, there are 36 to 78 nozzles distributed over the low-alloy steel head. The vessel head is semi-spherical and the head penetrations are vertical so that the CRD nozzles and other VHPs are not perpendicular to the vessel surface except at the center. The uphill side (toward the center of the head) is called the 180-degree location and the downhill side (toward the outer periphery of the head) is called the 0-degree location. Most nozzles have a thermal sleeve with a conical guide at the bottom end and a small gap (3- to 4-mm) between the nozzle and the sleeve.

The NRC staff identified primary water stress corrosion cracking (PWSCC) as an emerging technical issue to the Commission in 1989, after cracking was noted in Alloy 600 pressurizer heater sleeve penetrations at a domestic PWR facility. Other leaks have occurred since 1986 in several Alloy 600 pressurizer instrument nozzles at both domestic and foreign reactors from several different nuclear steam supply system vendors. The NRC staff reviewed the safety significance of the cracking that occurred, as well as the repair and replacement activities at the affected facilities. The NRC staff determined that the cracking was not of immediate safety significance because the cracks were axial, had a low growth rate, were in a material with an extremely high flaw tolerance (high fracture toughness) and, accordingly, were unlikely to propagate very far. These factors also demonstrated that any cracking would result in detectable leakage and the opportunity to take corrective action before a penetration would fail. The NRC staff issued Information Notice 90-10, "Primary Water Stress Corrosion Cracking (PWSCC) of Inconel 600," dated February 23, 1990, to inform the nuclear industry of the issue.

In December 1991, cracks were found in an Alloy 600 VHP in the reactor head at Bugey 3, a French PWR.

Examinations in PWRs in France, Belgium, Switzerland, Sweden, Spain, and Japan have uncovered additional VHPs with axial cracks. About 2 percent of the VHPs examined to date contain short, axial cracks. Close examination of the VHP that leaked at Bugey 3 revealed very minor incipient secondary circumferential cracking of the VHP.

An action plan was implemented by the NRC staff in 1991 to address PWSCC of Alloy 600 VHPs at all U.S. PWRs. As explained more fully below, this action plan included a review of the safety