

unless, upon application and for good cause shown, this date is extended.

This Order is effective upon issuance.

#### IV

By July 25, 1997, any person adversely affected by this Order may file a request for a hearing with respect to issuance of the Order. Any person requesting a hearing shall set forth with particularity how such person's interest is adversely affected by this Order and shall address the criteria set forth in 10 CFR 2.714(d).

If a hearing is to be held, the Commission will issue an order designating the time and place of such hearing.

The issue to be considered at any such hearing shall be whether this Order should be sustained.

Any request for a hearing must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Rulemakings and Adjudications Staff, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. Copies should also be sent to the Office of the General Counsel and to the Director, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to Gerald Charnoff, Esquire, of Shaw, Pittman, Potts and Trowbridge, 2300 N Street NW., Washington, DC 20037.

For further details with respect to this action, see the application dated December 13, 1996, as supplemented February 12, 1997, and the Safety Evaluation dated June 19, 1997, which are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the University of Toledo, William Carlson Library, Government Documents Collection, 2801 West Bancroft Avenue, Toledo, Ohio.

Dated at Rockville, Maryland, this 19th day of June 1997.

For The U.S. Nuclear Regulatory Commission.

**Frank J. Miraglia, Jr.,**

*Acting Director, Office of Nuclear Reactor Regulation.*

[FR Doc. 97-16614 Filed 6-24-97; 8:45 am]

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## NUCLEAR REGULATORY COMMISSION

[Docket No. 50-334]

### Duquesne Light Company, et al.; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an exemption from certain requirements of its regulations for Facility Operating License No. DPR-66, issued to Duquesne Light Company, et al. (the licensee), for operation of the Beaver Valley Power Station, Unit No. 1 (BVPS-1), located in Beaver County, Pennsylvania.

#### Environmental Assessment

##### Identification of Proposed Action

The proposed action would exempt the licensee from the requirements of 10 CFR 70.24, which require a monitoring system that will energize clear audible alarms if accidental criticality occurs in each area in which special nuclear material is handled, used, or stored. The proposed action would also exempt the licensee from the requirements to maintain emergency procedures for each area in which this licensed special nuclear material is handled, used, or stored to ensure that all personnel withdraw to an area of safety upon the sounding of the alarm, to familiarize personnel with the evacuation plan, and to designate responsible individuals for determining the cause of the alarm, and to place radiation survey instruments in accessible locations for use in such an emergency.

The proposed action is in accordance with the licensee's application for exemption dated December 18, 1996, as supplemented by letters dated April 10 and June 11, 1997.

##### The Need for the Proposed Action

The purpose of 10 CFR 70.24 is to ensure that if a criticality were to occur during the handling of special nuclear material, personnel would be alerted to that fact and would take appropriate action. At a commercial nuclear power plant the inadvertent criticality with which 10 CFR 70.24 is concerned could occur during fuel handling operations. The special nuclear material that could be assembled into a critical mass at a commercial nuclear power plant is in the form of nuclear fuel; the quantity of other forms of special nuclear material that is stored on site is small enough to preclude achieving a critical mass. Because the fuel is not enriched beyond a nominal 5.0 weight percent Uranium-

235 and because commercial nuclear plant licensees have procedures and design features that prevent inadvertent criticality, the staff has determined that an inadvertent criticality is highly unlikely as a result of the handling of special nuclear material at a commercial power reactor. The requirements of 10 CFR 70.24, therefore, are not necessary to ensure the safety of personnel during the handling of special nuclear materials at commercial power reactors.

##### Environmental Impacts of the Proposed Action

The Commission has completed its evaluation of the proposed action and concludes that there is no significant environmental impact if the exemption is granted. Inadvertent or accidental criticality will be precluded through compliance with the BVPS-1 Technical Specifications (TSs), the design of the fuel storage racks providing geometric spacing of fuel assemblies in their storage locations, and administrative controls imposed on fuel handling procedures. TSs requirements specify reactivity limits for the fuel storage racks and minimum spacing between the fuel assemblies in the storage racks.

Appendix A of 10 CFR Part 50, "General Design Criteria for Nuclear Power Plants," Criterion 62, requires that the criticality in the fuel storage and handling system be prevented by physical systems or processes, preferably by use of geometrically-safe configurations. This is met at BVPS-1, as identified in the TSs and the Updated Final Safety Analysis Report (UFSAR). BVPS-1 TS 5.3.1.2 states that the new fuel storage racks are designed and shall be maintained with a nominal 21-inch center-to-center distance between fuel assemblies placed in the storage racks. This spacing requirement ensures that  $k_{eff}$  will be  $\leq 0.95$  if the loaded new fuel storage racks are flooded with unborated water and that  $k_{eff}$  will be  $\leq 0.98$  if the loaded new fuel storage racks are moderated by aqueous foam. UFSAR Section 9.12.1.1 (Prevention of Fuel Storage Criticality) states that new fuel assemblies will be stored dry and vertically in the new fuel storage racks with a minimum center-to-center spacing of 21 inches.

The proposed exemption would not result in any significant radiological impacts. The proposed exemption would not affect radiological plant effluents nor cause any significant occupational exposures since the TSs, design controls (including geometric spacing of fuel assembly storage spaces) and administrative controls preclude inadvertent criticality. The amount of

radioactive waste would not be changed by the proposed exemption.

Accordingly, the Commission concludes that the proposed action would result in no significant radiological environmental impact.

The proposed exemption does not result in any significant nonradiological environmental impacts. The proposed exemption involves features located entirely within the restricted area as defined in 10 CFR Part 20. It does not affect non-radiological plant effluents and has no other environmental impact. Accordingly, the Commission concludes that there are no significant non-radiological environmental impacts associated with the proposed action.

#### *Alternative to the Proposed Action*

Since the Commission has concluded that there is no measurable environmental impact associated with the proposed action, any alternatives with equal or greater environmental impact need not be evaluated. As an alternative to the proposed exemption, the staff considered denial of the requested exemption. Denial of the request would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

#### *Alternative Use of Resources*

This action does not involve the use of any resources not previously considered in the Final Environmental Statement for the Beaver Valley Power Station, Unit No. 1, dated July 1973.

#### *Agencies and Persons Consulted*

In accordance with its stated policy, on June 3, 1997, the staff consulted with the Pennsylvania State official, Mr. Richard Janati of the Bureau of Radiation Protection, Department of Environmental Protection, regarding the environmental impact of the proposed action. The State official had no comments.

#### **Finding of No Significant Impact**

Based upon the environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated December 18, 1996, as supplemented April 10 and June 11, 1997, which is available for public inspection at the Commission's Public Document Room, which is located at

The Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the B. F. Jones Memorial Library, 663 Franklin Avenue, Aliquippa, Pennsylvania 15001.

Dated at Rockville, Maryland, this 18th day of June, 1997.

For The Nuclear Regulatory Commission.

**Chester Poslusny,**

*Acting Director, Project Directorate I-2,  
Division of Reactor Projects—I/II, Office of  
Nuclear Reactor Regulation.*

[FR Doc. 97-16611 Filed 6-24-97; 8:45 am]

BILLING CODE 7590-01-P

#### **NUCLEAR REGULATORY COMMISSION**

[Docket No. 50-409]

#### **Lacrosse Boiling Water Reactor; Closing of Local Public Document Room**

Notice is hereby given that the Nuclear Regulatory Commission (NRC) is closing the local public document room (LPDR) for records pertaining to the Dairyland Power Cooperative's LaCrosse Boiling Water Reactor (BWR) located at the LaCrosse Public Library, LaCrosse, Wisconsin, effective June 30, 1997.

The LaCrosse Public Library has served as the LPDR for the LaCrosse BWR for 25 years. In a letter dated February 14, 1997, the library director officially informed the NRC that they no longer wish to serve as the LPDR since there is no longer a demand for the document collection. NRC has made the decision to officially close the LaCrosse LPDR because none of the libraries in the vicinity of the facility are interested in maintaining the document collection, the facility has been shut down since 1987 and is in the SAFSTOR method of decommissioning, and there has been no demonstrated local public interest in the LPDR materials for a number of years. Therefore, effective June 30, 1997, the LPDR will be closed.

Persons now interested in information pertaining to this facility or any other NRC activity may contact the NRC Public Document Room by calling toll-free 1-800-397-4209 or writing to NRC Public Document Room, Washington, DC 20555-0001.

Dated at Rockville, Maryland, this 19th day of June, 1997.

For the Nuclear Regulatory Commission.

**Russell A. Powell,**

*Chief, Freedom of Information/Local Public  
Document Room Branch, Office of  
Information Resources Management.*

[FR Doc. 97-16616 Filed 6-24-97; 8:45 am]

BILLING CODE 7590-01-P

#### **NUCLEAR REGULATORY COMMISSION**

#### **Use of PRA in Plant Specific Reactor Regulatory Activities: Proposed Regulatory Guides, Standard Review Plan Sections, and Supporting NUREG**

**AGENCY:** Nuclear Regulatory  
Commission.

**ACTION:** Notice of availability.

**SUMMARY:** The Nuclear Regulatory Commission has issued for public comment drafts of four regulatory guides, three Standard Review Plan Sections, and a NUREG document. These issuances follow Publication of the Commission's August 16, 1995 (60 FR 42622) Policy statement on the Use of PRA Methods in Nuclear Regulatory Activities. The NRC has developed draft guidance for power reactor licensees on acceptable methods for using probabilistic risk assessment (PRA) information and insights in support of plant-specific applications to change the current licensing basis (CLB). The use of such PRA information and guidance is voluntary. To facilitate comment, the Commission intends to conduct a workshop during the comment period to explain the draft documents and answer questions. The exact time, location and agenda will be announced in a future issue of the **Federal Register**. Section VI of this notice provides additional information on the scope, purpose and topics for discussion at the workshop.

**DATES:** Comment period expires September 23, 1997. Comments received after this date will be considered if it is practical to do so, but the Commission is able to assure consideration only for comments received on or before this date.

**ADDRESSES:** Mail written comments to: David L. Meyer, Chief, Rules and Directives Branch, Office of Administration, Mail Stop T-6D59, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

In addition to written comments, please (1) attach a diskette containing your comments, in either ASCII text or Wordperfect format (Version 5.1 or 6.1), or (2) submit your comments electronically via the NRC Electronic Bulletin Board on FedWorld or the NRC's Interactive Rulemaking Website.