became totally or partially separated from employment on or after December 28, 1997 through March 11, 2001 are eligible to apply for NAFTA-TA under section 250 of the Trade Act of 1974.

Signed at Washington, DC, this 27th day of May, 1999.

Grant D. Beale,

Acting Director, Office of Trade Adjustment Assistance.

[FR Doc. 99-15304 Filed 6-15-99; 8:45 am] BILLING CODE 4510-30-M

DEPARTMENT OF LABOR

Occupational Safety and Health Administration

[Docket No. H-372]

RIN 1218-AB58

Metalworking Fluids Standards Advisory Committee: Notice of Meeting

AGENCY: Occupational Safety and Health Administration (OSHA), Labor.

ACTION: Metalworking Fluids Standards Advisory Committee: Notice of meeting.

SUMMARY: The Metalworking Fluids Standards Advisory Committee (MWFSAC), established under section 7 of the Occupational Safety and Health Act of 1970 to advise the Secretary of Labor on appropriate actions to protect workers from the hazards associated with occupational exposure to metalworking fluids, will meet in Washington, DC on Wednesday, July 7, Thursday, July 8 and Friday, July 9,

DATES: The meeting will be held July 7. 1999 from 10 a.m. to 6 p.m.; on July 8 from 8 a.m. to 5 p.m.; and on July 9 from 8 a.m. to 3 p.m.

ADDRESSES: The Committee will meet at the Capital Hilton Hotel, 16th & K Streets, NW, Washington, DC 20036, Telephone: 202-393-1000.

Mail comments, views, or statements in response to this notice to Dr. Peter Infante, U.S. Department of Labor, OSHA, Directorate of Health Standards Programs, Metalworking Fluids Standards Advisory Committee, Room N-3718, 200 Constitution Avenue, NW, Washington, DC 20210.

FOR FURTHER INFORMATION CONTACT: Bonnie Friedman, Director, Office of Public Affairs, OSHA, (202) 693-1999.

SUPPLEMENTARY INFORMATION: All

interested persons are invited to attend the public meetings of the MWFSAC at the times and location indicated above. Individuals with disabilities wishing to attend should contact Theresa Berry at (202) 693-1999 (Fax: 202-693-1634) no later than June 28, 1999, to obtain appropriate accommodations.

Meeting Agenda

The MWFSAC will discuss its draft final report to OSHA. The report is expected to include the Committee's recommendations for OSHA action and best practices for working in the metalworking fluid environment, including medical surveillance, training, and exposure monitoring. The Committee intends to complete and ratify its final report to OSHA at the meeting. OSHA will present a risk assessment update and report on data collected by the Michigan Occupational Safety and Health Administration (MIOSHA).

Public Participation

Written data, views, or comments for consideration by the MWFSAC on the various agenda items listed above may be submitted, preferably with 25 copies, to Dr. Peter Infante. Submissions received by June 28, 1999, will be provided to the members of the Committee. Anyone wishing to make an oral presentation to the Committee on any of the agenda items listed above should notify Dr. Peter Infante at the address listed above. The request to speak should state the amount of time desired, the capacity in which the person will appear, and a brief outline of the content of the presentation. Requests to make oral presentations to the Committee may be granted if time permits.

Authority: This notice is issued under the authority of sections 6(b)(1) and 7(b) of the Occupational Safety and Health Act of 1970 (29 U.S.C. 655, 656), the Federal Advisory Committee Act (5 U.S.C. App. 2), and 29 CFR part 1912.

Signed at Washington, D.C. this 10th day of June, 1999.

Charles N. Jeffress,

Assistant Secretary of Labor.

[FR Doc. 99-15239 Filed 6-15-99; 8:45 am] BILLING CODE 4510-26-P

NUCLEAR REGULATORY COMMISSION

Agency Information Collection Activities: Submission for OMB Review; Comment Request

AGENCY: U.S. Nuclear Regulatory Commission (NRC).

ACTION: Notice of the OMB review of information collection and solicitation of public comment.

SUMMARY: The NRC has recently submitted to OMB for review the following proposal for the collection of information under the provisions of the Paperwork Reduction Act of 1995 (44

- U.S.C. Chapter 35). The NRC hereby informs potential respondents that an agency may not conduct or sponsor, and that a person is not required to respond to, a collection of information unless it displays a currently valid OMB control number.
- 1. Type of submission, new, revision, or extension: Revision.
- 2. The title of the information collection:

NRC Form 313, "Application for Material License.

NRC Form 313A, "Training and Experience"

NRC Form 313B, "Preceptor Statement"

3. The form number if applicable:

NRC Form 313 NRC Form 313A

NRC Form 313B

- 4. How often the collection is required: There is a one-time submittal of information to receive a license. Once a specific license has been issued, there is a 10-year resubmittal of the information for renewal of the license. Amendments are submitted as needed by the licensee.
- 5. Who will be required or asked to report: All applicants requesting a license, and licensees requesting renewal or amendment of a byproduct or source material license to possess, use, or distribute radioactive material.
- 6. An estimate of the number of responses: 9007 (2522 NRC licensees and 6485 Agreement State licensees)
- 7. The estimated number of annual respondents: 17,958 (5,556 NRC licensees and 12,402 Agreement State licensees) This is the total number of licensees which could potentially submit licensing actions.
- 8. An estimate of the total number of hours needed annually to complete the requirement or request: 66,652 (18,663 hours for NRC licensees and 47,989 hours for Agreement State licensees, an average of about 7.4 hours per response).
- 9. An indication of whether Section 3507(d), Pub. L. 104-13 applies: Not applicable.
- 10. Abstract: All applicants must submit NRC Form 313 to obtain, renew, or amend a specific license to possess, use, or distribute byproduct or source material. NRC Form 313A, "Training and Experience," and NRC Form 313B, "Preceptor Statement," are used for 10 CFR Part 35, "Medical Use of Byproduct Material," applicants and licensees along with NRC Form 313 to obtain the above information. The information is reviewed by the NRC to determine whether the applicant is qualified by training and experience, and has equipment, facilities, and procedures

which are adequate to protect the health and safety of the public, and minimize

danger to life or property.

A copy of the final supporting statement may be viewed free of charge at the NRC Public Document Room, 2120 L Street, NW (lower level), Washington, DC. OMB clearance requests are available at the NRC worldwide web site (http://www.nrc.gov/NRC/PUBLIC/OMB/index.html). The document will be available on the NRC home page site for 60 days after the signature date of this notice.

Comments and questions should be directed to the OMB reviewer listed below by July 16, 1999. Comments received after this date will be considered if it is practical to do so, but assurance of consideration cannot be given to comments received after this date.

Erik Godwin, Office of Information and Regulatory Affairs (3150–0120), NEOB–10202, Office of Management and Budget, Washington, DC 20503. Comments can also be submitted by telephone at (202) 395–3087.

The NRC Clearance Officer is Brenda Jo. Shelton, 301–415–7233.

Dated at Rockville, Maryland, this 10th day of June 1999.

For the Nuclear Regulatory Commission. **Brenda Jo. Shelton**,

NRC Clearance Officer, Office of the Chief Information Officer.

[FR Doc. 99–15242 Filed 6–15–99; 8:45 am] BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. STN 50-528, STN 50-529, and STN 50-530]

Arizona Public Service Company; Palo Verde Nuclear Generating Station, Units 1, 2, and 3; Notice of Withdrawal of Application for Amendments to Facility Operating Licenses

The U.S. Nuclear Regulatory Commission (the Commission) has granted the request of Arizona Public Service Company (the licensee) to withdraw its November 6, 1996, application for proposed amendments to Facility Operating Licenses Nos. NPF– 41, NPF–51, and NPF–74, for the Palo Verde Nuclear Generating Station (Palo Verde), Units 1, 2, and 3, located in Maricopa County, Arizona.

The proposed amendments would have revised the facility technical specifications to provide a method to respond to a sustained, degraded switchyard voltage condition.

The Commission had previously issued a Notice of Consideration of Issuance of Amendments published in the **Federal Register** on January 2, 1997 (62 FR 123). However, by letter dated December 16, 1998, the licensee withdrew the proposed change.

For further details with respect to this action, see the application for amendments dated November 6, 1996, and the licensee's letter dated December 16, 1998, which withdrew the application for license amendments. The above documents are available for public inspection—2 -at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Phoenix Public Library, 1221 N. Central Avenue, Phoenix, Arizona 85004.

Dated at Rockville, Maryland, this 7th day of June 1999.

For the Nuclear Regulatory Commission. **Mel B. Fields**,

Project Manager, Section 2, Project Directorate IV & Decommissioning Division of Licensing Project Management Office of Nuclear Reactor Regulation

[FR Doc. 99-15243 Filed 6-15-99; 8:45 am] BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. STN 50-454, STN 50-455, STN 50-456 and STN 50-457]

Commonwealth Edison Company; Notice of Consideration of Issuance of Amendments to Facility Operating Licenses, Proposed No Significant Hazards Consideration Determination and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of amendments to Facility Operating License Nos. NPF–37 and NPF–66 issued to the Commonwealth Edison Company (ComEd, the licensee) for operation of Byron Station, Unit Nos. 1 and 2, respectively, located in Ogle County, Illinois, and Facility Operating License Nos. NPF–72 and NPF–77 issued to ComEd for the operation of Braidwood Station, Unit Nos. 1 and 2, respectively, located in Will County, Illinois.

The proposed amendments would change the Technical Specifications to support a plant modification to install new storage racks for fuel in the spent fuel pools (SFP). As part of the modification, the total capacity of the SFP at each station is being increased from 2,870 assemblies to 2,984 assemblies.

Before issuance of the proposed license amendments, the Commission will have made findings as required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendments requested involve no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendments would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

The proposed Technical Specifications (TS) changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

During the installation of the new Holtec spent fuel pool storage racks, both Holtec and the existing Joseph Oat spent fuel pool storage racks will be in the spent fuel pool at the same time. This interim arrangement will not increase the probability or consequences of an accident previously evaluated. The criticality analysis for the Joseph Oat spent fuel pool storage racks states that should a spent fuel pool water temperature change accident or a fuel assembly misload accident occur in the Region 1, Region 2, or failed fuel storage cells, keff will be maintained less than or equal to 0.95 due to the presence of at least 550 ppm (no fuel handling) or 1650 ppm (during fuel handling) of soluble boron in the spent fuel pool water. These assumptions are more conservative than the requirements stated in the criticality analysis for the Holtec spent fuel pool storage racks which only requires 220 ppm boron to maintain k_{eff} less than or equal to 0.95 during the worst case fuel assembly misload accident. The new Holtec racks have a superior neutron attenuation capability due to their improved design. The requirement of 2000 ppm boron will be maintained during the entire change out process, therefore, ensuring that keff will remain less than or equal to 0.95. At the completion of installation, only Holtec spent fuel pool storage racks will be in the spent fuel pool.

The previously evaluated Byron and Braidwood Stations accidents relative to spent fuel storage are discussed in the Updated Final Safety Analysis Report (UFSAR) Section 15.7.4, "Fuel Handling Accidents," and UFSAR Section 15.7.5, "Spent Fuel Cask Drop Accident." These accidents were considered for the new Holtec spent fuel pool racks and are listed below.

 a. Spent fuel assembly dropped onto the spent fuel pool floor.