

According to an application for approval filed by FPC, the City and FPC have reached an agreement that provides for the transfer of the City's 1.3333 percent ownership interest in CR-3 to FPC in exchange for FPC assuming responsibility for certain future liabilities. FPC presently owns about 90 percent of CR-3, and is exclusively authorized under the license to operate, maintain, and decommission the facility. No physical changes to, or operational changes for CR-3 are being proposed in the application. Also, no changes to FPC's authority to operate, maintain, and decommission the facility are being requested. The application seeks, in addition to the Commission's consent to the transfer, approval of a license amendment to remove the City from the license once the transfer is approved and completed.

Pursuant to 10 CFR 50.80, no license, or any right thereunder, shall be transferred, directly or indirectly, through transfer of control of the license, unless the Commission shall give its consent in writing. The Commission will approve an application for the transfer of a license, if the Commission determines that the proposed transferee is qualified to hold the license, and that the transfer is otherwise consistent with applicable provisions of law, regulations, and orders issued by the Commission pursuant thereto.

Before issuance of the proposed conforming license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations.

As provided in 10 CFR 2.1315, unless otherwise determined by the Commission with regard to a specific application, the Commission has determined that any amendment to the license of a utilization facility which does no more than conform the license to reflect the transfer action involves no significant hazards consideration. No contrary determination has been made with respect to this specific license amendment application. In light of the generic determination reflected in 10 CFR 2.1315, no public comments with respect to significant hazards considerations are being solicited, notwithstanding the general comment procedures contained in 10 CFR 50.91.

The filing of requests for a hearing and petitions for leave to intervene, and written comments with regard to the license transfer application, are discussed below.

By March 18, 1999, any person whose interest may be affected by the Commission's action on the application

may request a hearing, and, if not the applicants, may petition for leave to intervene in a hearing proceeding on the Commission's action. Requests for a hearing and petitions for leave to intervene should be filed in accordance with the Commission's rules of practice set forth in Subpart M, "Public Notification, Availability of Documents and Records, Hearing Requests and Procedures for Hearings on License Transfer Applications," of 10 CFR Part 2. In particular, such requests and petitions must comply with the requirements set forth in 10 CFR 2.1306, and should address the considerations contained in 10 CFR 2.1308(a). Untimely requests and petitions may be denied, as provided in 10 CFR 2.1308(b), unless good cause for failure to file on time is established. In addition, an untimely request or petition should address the factors that the Commission will also consider, in reviewing untimely requests or petitions, set forth in 10 CFR 2.1308(b)(1)-(2).

Requests for a hearing and petitions for leave to intervene should be served upon R. Alexander Glenn, General Counsel, Florida Power Corporation, MAC-A5A, P. O. Box 14042, St. Petersburg, Florida 33733-4042; the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555; and the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Rulemakings and Adjudications Staff, in accordance with 10 CFR 2.1313.

The Commission will issue a notice or order granting or denying a hearing request or intervention petition, designating the issues for any hearing that will be held and designating the Presiding Officer. A notice granting a hearing will be published in the **Federal Register** and served on the parties to the hearing.

As an alternative to requests for a hearing and petitions to intervene, by March 29, 1999, persons may submit written comments regarding the license transfer application, as provided for in 10 CFR 2.1305. The Commission will consider and, if appropriate, respond to these comments, but such comments will not otherwise constitute part of the decisional record. Comments should be submitted to the Secretary, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Rulemakings and Adjudications Staff, and should cite the publication date and page number of this **Federal Register** notice.

For further details with respect to this action, see the application dated December 29, 1998, available for public

inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Coastal Region Library, 8619 W. Crystal Street, Crystal River, Florida 34428.

Dated at Rockville, Maryland this 19th day of February 1999.

For the Nuclear Regulatory Commission.

**Cecil O. Thomas,**

*Director, Project Directorate II-3, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.*

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## NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-361 and 50-362]

### Southern California Edison Company; Notice of Withdrawal of Application for Amendments to Facility Operating Licenses

The U.S. Nuclear Regulatory Commission (the Commission) has granted the request of Southern California Edison Company, *et al.* (the licensee) to withdraw its December 30, 1992, application for proposed amendments to Facility Operating License Nos. NPF-10 and NPF-15 for the San Onofre Nuclear Generating Station, Unit Nos. 2 and 3, located in San Diego County, California.

The proposed change would have modified Technical Specifications 3/4.3.2, "Engineered Safety Feature Actuation System Instrumentation," and 3/4.3.3, "Radiation Monitoring Instrumentation" to eliminate the technical specification requirements and engineered safety feature actuation system circuitry for the control room isolation system particulate/iodine channel.

The Commission had previously issued a Notice of Consideration of Issuance of Amendments published in the **Federal Register** on March 3, 1993 (58 FR 12267). However, by letter dated August 11, 1995, the licensee withdrew the amendments request indicating that it had been superseded by the technical specification improvement program application dated December 30, 1993.

For further details with respect to this action, see the application for amendments dated December 30, 1992, and the licensee's letter dated August 11, 1995, which withdrew the application for license amendment. The above documents are available for public inspection at the Commission's Public Document Room, the Gelman

Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Main Library, University of California, P.O. Box 19557, Irvine, California 92713.

Dated at Rockville, Maryland, this 22nd day of February 1999.

For the Nuclear Regulatory Commission.

**James W. Clifford,**

*Senior Project Manager, Project Directorate IV-2, Division of Reactor Projects III/IV, Office of Nuclear Reactor Regulation.*

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## NUCLEAR REGULATORY COMMISSION

[Docket No. 50-482]

### Wolf Creek Nuclear Operating Corporation; Notice of Consideration of Issuance of Amendment to Facility Operating License and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF-42, issued to the Wolf Creek Nuclear Operating Corporation (WCNOC or the licensee), for operation of the Wolf Creek Generating Station (WCGS), located in Coffey County, Kansas.

The initial Notice of Consideration of Issuance of Amendment to Facility Operating License and Opportunity for Hearing was published in the **Federal Register** on October 5, 1998 (63 FR 53471). The information included in the supplemental letters indicates that the original notice, that included fourteen proposed beyond-scope issues (BSIs) to the Improved Technical Specifications (ITS) conversion, needs to be expanded to add sixteen new BSIs and revised to delete 8 previous BSIs. This includes a total of twenty-two BSIs.

The proposed amendment, requested by the licensee in a letter dated May 15, 1997, as supplemented by letters dated June 30, August 5, August 28, September 24, October 16, October 23, November 24, December 2, December 17, December 21, 1998 and February 4, 1999, would represent a full conversion from the current Technical Specifications (CTS) to a set of improved Technical Specifications (ITS) based on NUREG-1431, "Standard Technical Specifications, Westinghouse Plants," Revision 1, dated April 1995. NUREG-1431 has been developed by the Commission's staff through working groups composed of both NRC staff members and industry representatives, and has been endorsed by the staff as

part of an industry-wide initiative to standardize and improve the Technical Specifications for nuclear power plants. As part of this submittal, the licensee has applied the criteria contained in the Commission's "Final Policy Statement on Technical Specification Improvements for Nuclear Power Reactors (Final Policy Statement)," published in the **Federal Register** on July 22, 1993 (58 FR 39132), to the CTS, and, using NUREG-1431 as a basis, proposed an ITS for WCGS. The criteria in the Final Policy Statement were subsequently added to 10 CFR 50.36, "Technical Specifications," in a rule change that was published in the **Federal Register** on July 19, 1995 (60 FR 36953) and became effective on August 18, 1995.

This conversion is a joint effort in concert with three other utilities: Pacific Gas & Electric Company for Diablo Canyon Power Plant, Units 1 and 2 (Docket Nos. 50-275 and 323); TU Electric for Comanche Peak Steam Electric Station, Units 1 and 2 (Docket Nos. 50-445 and 50-446); and Union Electric Company for Callaway Plant (Docket No. 50-483). It is a goal of the four utilities to make the ITS for all the plants as similar as possible. This joint effort includes a common methodology for the licensees in marking-up the CTS and NUREG-1431 Specifications, and the NUREG-1431 Bases, that has been accepted by the staff. This includes the convention that, if the words in the CTS specification are not the same as the words in the ITS specification but they mean the same or have the same requirements as the words in the ITS specification, the licensee does not indicate or describe the change to the CTS.

This common methodology is discussed at the end of Enclosure 2, "Mark-Up of Current TS"; Enclosure 5a, "Mark-Up of NUREG-1431 Specifications"; and Enclosure 5b, "Mark-Up of NUREG-1431 Bases, for each of the 14 separate ITS sections that were submitted with the licensee's application. For each of the 14 ITS sections, there is also the following: Enclosure 1, the cross reference table connecting each CTS specification (i.e., limiting condition for operation, required action, or surveillance requirement) to the associated ITS specification, sorted by both CTS and ITS Specifications; Enclosure 3, the description of the changes to the CTS section and the comparison table showing which plants (of the four licensees in the joint effort) that each change applies to; Enclosure 4, the no significant hazards consideration (NHSC) of 10 CFR 50.91 for the changes

to the CTS with generic NHSCs for administrative, more restrictive, relocation, and moving-out-of-CTS changes, and individual NHSCs for less restrictive changes and with the organization of the NHSC evaluation discussed in the beginning of the enclosure; and Enclosure 6, the descriptions of the differences from NUREG-1431 specifications and the comparison table showing which plants (of the four licensees in the joint effort) that each difference applies to. Another convention of the common methodology is that the technical justifications for the less restrictive changes are included in the NHSCs.

The licensee has categorized the proposed changes to the CTS into four general groupings. These groupings are characterized as administrative changes, relocated changes, more restrictive changes and less restrictive changes.

Administrative changes are those that involve restructuring, renumbering, rewording, interpretation and complex rearranging of requirements and other changes not affecting technical content or substantially revising an operating requirement. The reformatting, renumbering and rewording process reflects the attributes of NUREG-1431 and does not involve technical changes to the existing TS. The proposed changes include (a) providing the appropriate numbers, etc., for NUREG-1431 bracketed information (information that must be supplied on a plant-specific basis, and which may change from plant to plant), (b) identifying plant-specific wording for system names, etc., and (c) changing NUREG-1431 section wording to conform to existing licensee practices. Such changes are administrative in nature and do not impact initiators of analyzed events or assumed mitigation of accident or transient events.

Relocated changes are those involving relocation of requirements and surveillances for structures, systems, components, or variables that do not meet the criteria for inclusion in TS. Relocated changes are those current TS requirements that do not satisfy or fall within any of the four criteria specified in the Commission's policy statement and may be relocated to appropriate licensee-controlled documents. There will be a license condition to require the licensee to implement the relocations as described in its letters.

The licensee's application of the screening criteria is described in Attachment 2 to its June 2, 1997, submittal, which is entitled, "General Description and Assessment." The affected structures, systems, components or variables are not