

and former workers of the subject firm. The denial notice was signed on May 3, 2002, and was published in the **Federal Register** on May 17, 2002 (67 FR 35142).

Pursuant to 29 CFR 90.18(c) reconsideration may be granted under the following circumstances:

(1) If it appears on the basis of facts not previously considered that the determination complained of was erroneous;

(2) If it appears that the determination complained of was based on a mistake in the determination of facts not previously considered; or

(3) If in the opinion of the Certifying Officer, a misinterpretation of facts or of the law justified reconsideration of the decision.

The denial of NAFTA-TAA for workers engaged in activities related to the production of carbon graphite machined products and raw graphite materials at Ividen Graphite of America Corporation, Portland, Oregon was based on the finding that criteria (3) and (4) of the group eligibility requirements of paragraph (a)(1) of Section 250 of the Trade Act, as amended, were not met. There were no company imports of carbon graphite machined products and raw graphite materials from Mexico or Canada, nor did the subject firm shift production from Portland, Oregon to Mexico or Canada. The survey conducted by the Department of Labor revealed no imports of carbon graphite machined products and raw graphite materials from Canada or Mexico during the relevant period.

The petitioner appears to be alleging that the production performed by the subject firm is now being sent to a facility located in Boston, Massachusetts, who then send most of that work to Japan.

The alleged shifts in production to a foreign source other than Canada or Mexico or the imports from a foreign source other than Canada or Mexico are not relevant factors in meeting the eligibility requirements for NAFTA-TAA under Section 250 of the Trade Act of 1974.

Of note, on June 28, 2002 the workers were certified eligible to apply for Trade Adjustment Assistance under TA-W-41,424.

Conclusion

After review of the application and investigative findings, I conclude that there has been no error or misinterpretation of the law or of the facts which would justify reconsideration of the Department of Labor's prior decision. Accordingly, the application is denied.

Signed at Washington, DC this 12th day of July 2002.

Edward A. Tomchick,

Director, Division of Trade Adjustment Assistance.

[FR Doc. 02-18426 Filed 7-19-02; 8:45 am]

BILLING CODE 4510-30-P

DEPARTMENT OF LABOR

Employment and Training Administration

[NAFTA-6286]

Joy Mining Machinery, Mt. Vernon, IL; Notice of Termination of Investigation

Pursuant to Title V of the North American Free Trade Agreement Implementation Act (Pub. L. 103-182) concerning transitional adjustment assistance, hereinafter called (NAFTA-TAA), and in accordance with Section 250(a), Subchapter D, Chapter 2, Title II, of the Trade Act of 1974, as amended (19 U.S.C. 2273), an investigation was initiated on June 12, 2002, in response to a petition filed by the International Brotherhood of Boilermakers, Iron Ship Builders, Blacksmiths, Forgers and Helpers, Local 483, on behalf of workers at Joy Mining Machinery, Mt. Vernon, Illinois (NAFTA-6286).

The date of the petition is June 3, 2002, however, the employees in question were terminated from employment November 24, 2000. Thus, all workers were separated from the subject firm more than one year prior to the date of the petition. Section 223(b) of the Act specifies that no certification may apply to any worker whose last separation occurred more than one year prior to the date of the petition.

Consequently, further investigation in this case would serve no purpose, and the investigation has been terminated.

Signed in Washington, DC this 10th day of July, 2002.

Edward A. Tomchick,

Director, Division of Trade Adjustment Assistance.

[FR Doc. 02-18428 Filed 7-19-02; 8:45 am]

BILLING CODE 4510-30-P

DEPARTMENT OF LABOR

Employment and Training Administration

[NAFTA-6322]

Whisper Jet, Inc., Sanford, FL; Notice of Termination of Investigation

Pursuant to Title V of the North American Free Trade Agreement Implementation Act (Pub.L. 103-182)

concerning transitional adjustment assistance, hereinafter called (NAFTA-TAA), and in accordance with Section 250(a), Subchapter D, Chapter 2, Title II, of the Trade Act of 1974, as amended (19 U.S.C. 2273), an investigation was initiated on June 20, 2002, in response to a petition filed on behalf of workers at Whisper Jet, Inc., Sanford, Florida.

The petitioner has requested that the petition be withdrawn. Consequently, further investigation in this case would serve no purpose, and the investigation has been terminated.

Signed at Washington, DC, this 12th day of July, 2002.

Linda G. Poole,

Certifying Officer, Division of Trade Adjustment Assistance.

[FR Doc. 02-18429 Filed 7-19-02; 8:45 am]

BILLING CODE 4510-30-P

NATIONAL CREDIT UNION ADMINISTRATION

Sunshine Act; Notice of Meeting

TIME AND DATE: 10 a.m. Thursday, July 25, 2002.

PLACE: Board Room, 7th Floor, Room 7047, 1775 Duke Street, Alexandria, VA 22314-3428.

STATUS: Open.

MATTERS TO BE CONSIDERED:

1. Quarterly Insurance Fund Report.
2. Reprogramming of NCUA's Operating Budget for 2002.
3. Request from a Federal Credit Union to Convert to a Community Charter.

FOR FURTHER INFORMATION CONTACT: Becky Baker, Secretary of the Board, Telephone: 703-518-6304.

Sheila Albin,

Acting Secretary of the Board.

[FR Doc. 02-18568 Filed 7-18-02; 1:13 pm]

BILLING CODE 7535-01-M

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-282 and 50-306]

Nuclear Management Company, LLC; Prairie Island Nuclear Generating Plant, Units 1 and 2; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of amendments to Facility Operating License Nos. DPR-42 and DPR-60, issued to Nuclear Management Company, LLC (the licensee) for

operation of the Prairie Island Nuclear Generating Plant, Units 1 and 2, located in Goodhue County, Minnesota.

Pursuant to title 10 of the Code of Federal Regulations (10 CFR) sections 51.21 and 51.32, the NRC is issuing this environmental assessment and finding of no significant impact.

Environmental Assessment

Identification of the Proposed Action

The proposed action would be a full conversion from the current technical specifications (CTS) to a set of improved technical specifications (ITS) based on NUREG-1431, "Standard Technical Specifications, Westinghouse Plants," Revision 1, dated April 1995. The proposed action is in accordance with the licensee's application dated December 11, 2000, as supplemented by letters dated March 6, July 3, August 13, November 12, and December 12, 2001, and January 25, January 31, February 14, February 15, February 16, March 6, April 11, May 10, May 30, June 7, June 25, and June 28, 2002.

The Need for the Proposed Action

The Commission's "Proposed Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" (52 FR 3788), dated February 6, 1987, contained an Interim Policy Statement that set forth objective criteria for determining which regulatory requirements and operating restrictions should be included in the technical specifications (TS). When it issued the Interim Policy Statement, the Commission also requested comments on it. Subsequently, to implement the Interim Policy Statement, each reactor vendor owners group and the NRC staff began developing standard TS (STS) for reactors supplied by each vendor. The Commission then published its "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" (58 FR 39132), dated July 22, 1993, in which it addressed comments received on the Interim Policy Statement, and incorporated experience in developing the STS. The Final Policy Statement formed the basis for a revision to 10 CFR 50.36 (60 FR 36953), dated July 19, 1995, that codified the criteria for determining the content of TS. The NRC Committee to Review Generic Requirements reviewed the STS, made note of their safety merits, and indicated its support of conversion by operating plants to the STS. For the Prairie Island Nuclear Generating Plant, Units 1 and 2, the STS are NUREG-1431, "Standard Technical Specifications, Westinghouse Plants," Revision 1, dated April 1995.

This document formed the basis for the Prairie Island Nuclear Generating Plant, Units 1 and 2, conversion.

The proposed changes to the CTS are based on NUREG-1431 and guidance provided in the Final Policy Statement. The objective of this action is to completely rewrite, reformat, and streamline the CTS (i.e., to convert the CTS to ITS). Emphasis was placed on human factors principles to improve clarity and understanding. The Bases section has been significantly expanded to clarify and better explain the purpose and foundation of each specification. In addition to NUREG-1431, portions of the CTS were also used as the basis for the development of the Prairie Island Nuclear Generating Plant, Units 1 and 2 ITS. Plant-specific issues (i.e., unique design features, requirements, and operating practices) were discussed at length with the licensee.

The proposed changes from the CTS can be categorized into five general groupings. These groupings are characterized as administrative changes, more restrictive changes, less restrictive changes, less restrictive relocated details, and relocated specifications. Administrative changes include those changes that are editorial in nature or involve the reorganization or reformatting of CTS requirements without affecting technical content or operational restrictions.

More restrictive changes include those changes that result in added restrictions or reduced flexibility. The licensee, in electing to implement the specifications of the STS, proposed a number of requirements more restrictive than those in the CTS. The ITS requirements in this category include requirements that are either new, more conservative than corresponding requirements in the CTS, or have additional restrictions that are not in the CTS but are in the STS.

Less restrictive changes include deletions and relaxations to portions of the CTS in order to conform to the guidance of NUREG-1431, which would result in reduced restrictions or added flexibility. When requirements have been shown to provide little or no safety benefit, their relaxation or removal from the TSs may be appropriate. In most cases, relaxations previously granted to individual plants on a plant-specific basis were the result of (1) generic NRC actions, (2) new staff positions that have evolved from technological advancements and operating experience, or (3) resolution of the Owner's Groups' comments on STS.

Less restrictive relocated details include those changes to the CTS that eliminate details and relocate the details

to licensee-controlled documents. Typically, this involves details of system designs, system descriptions including design limits, descriptions of system or plant operation, procedural details for meeting TS requirements and relocated reporting requirements, and redundant requirement references.

Relocated specifications include those changes to the CTS that relocate certain requirements which do not meet the 10 CFR 50.36 selection criteria. These requirements may be relocated to the Bases, updated safety analysis report, core operating limits report (COLR), operational quality assurance plan, plant procedures, or other licensee-controlled documents. Relocating requirements to licensee-controlled documents does not eliminate them, but rather, places them under more appropriate regulatory controls (i.e., 10 CFR 50.54(a)(3), and 10 CFR 50.59) to manage their implementation and future changes.

In addition to the proposed changes solely involving the conversion, there are also changes proposed that are (1) different from the requirements in both the CTS and the STS and, (2) in addition to those changes that are needed to meet the overall purpose of the conversion. These changes are referred to as beyond-scope changes and include:

1. Extension of the certain surveillance interval from 18 months to 24 months to support the proposed refueling cycle of 24 months;
2. Extension of the allowed outage time for the emergency core cooling system accumulators from 1 to 24 hours;
3. Missed surveillance consolidated line item improvement to extend the delay period for a missed surveillance requirement from the current limit of 24 hours to " * * * up to 24 hours or up to the limit of the specified Frequency, whichever is greater;"
4. Revision to the ventilation filter testing program to incorporate the guidance provided in NRC Generic Letter 99-02, "Laboratory Testing of Nuclear-Grade Activated Charcoal," dated June 3, 1999;
5. A new methodology (to be incorporated by reference into ITS Section 5.0) that describes the method by which the shutdown margin limit during physics testing is established for inclusion within the COLR;
6. A new methodology (to be incorporated by reference to ITS Section 5.0) that describes the method by which a factor, F_{QA} , (in support of ITS 3.2.1, Heat Flux Channel Factor) is to be determined; and
7. Plant-specific instrument setpoint methodology in support of new

instrument allowable values and trip setpoints in the ITS.

Environmental Impacts of the Proposed Action

The NRC staff has completed its evaluation of the proposed action and concludes that the proposed TS conversion would not increase the probability or consequences of accidents previously analyzed and would not affect facility radiation levels or facility radiological effluents. Specifically, the proposed TS changes will not increase the probability or consequences of accidents, no changes are being made in the types or amounts of any effluent that may be released offsite, and there is no significant increase in the allowable individual or cumulative occupational radiation exposure. Therefore, there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential nonradiological impacts, the proposed action does not have a potential to affect any historic sites because no previously undisturbed area will be affected by the proposed TS changes. It does not affect nonradiological plant effluents and has no other environmental impact. Therefore, there are no significant nonradiological environmental impacts associated with the proposed action.

Accordingly, the NRC concludes that there are no significant environmental impacts associated with the proposed action.

Environmental Impacts of the Alternatives to the Proposed Action

As an alternative to the proposed action, the staff considered denial of the proposed action (i.e., the "no-action" alternative). Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

The action does not involve the use of any different resources than those previously considered in the Final Environmental Statement for the Prairie Island Nuclear Generating Plant, Units 1 and 2, dated May 1973.

Agencies and Persons Consulted

On July 8, 2002, the staff consulted with Ms. Linda Bruemmer of Minnesota State Division of Environmental Health regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

On the basis of the environmental assessment, the NRC concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the NRC has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated December 11, 2000, as supplemented by letters dated March 6, July 3, August 13, November 12, and December 12, 2001, and January 25, January 31, February 14, February 15, February 16, March 6, April 11, May 10, May 30, June 7, June 25, and June 28, 2002. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room (PDR), located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the Agencywide Documents Access and Management System (ADAMS) Public Electronic Reading Room on the internet at the NRC Web site, <http://www.nrc.gov/reading-rm/adams/adams.html>. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS, should contact the NRC PDR Reference staff by telephone at 1-800-397-4209 or 301-415-4737, or by e-mail to pdr@nrc.gov.

Dated at Rockville, Maryland, this 15th day of July 2002.

For the Nuclear Regulatory Commission.

L. Raghavan,

Chief, Section 1, Project Directorate III, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 02-18434 Filed 7-19-02; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Draft Regulatory Guide; Issuance, Availability

The Nuclear Regulatory Commission has issued for public comment a proposed guide in its Regulatory Guide Series. Regulatory Guides are developed to describe and make available to the public such information as methods acceptable to the NRC staff for implementing specific parts of the NRC's regulations, techniques used by the staff in evaluating specific problems or postulated accidents, and data needed by the staff in its review of applications for permits and licenses.

The draft guide is temporarily identified by its task number, DG-1099,

which should be mentioned in all correspondence concerning this draft guide. Draft Regulatory Guide DG-1099, "Anchoring Components and Structural Supports in Concrete," is being developed to provide guidance to licensees and applicants on methods acceptable to the NRC staff for complying with the NRC's regulations on the design, evaluation, and quality assurance of anchors (steel embeddings) used for component and structural supports on concrete structures.

This draft guide has not received complete staff approval and does not represent an official NRC staff position.

Comments may be accompanied by relevant information or supporting data. Written comments may be submitted by mail to the Rules and Directives Branch, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555; or they may be hand-delivered to the Rules and Directives Branch, ADM, at 11555 Rockville Pike, Rockville, MD. Copies of comments received may be examined at the NRC Public Document Room, 11555 Rockville Pike, Rockville, MD. Comments will be most helpful if received by October 25, 2002.

You may also provide comments via the NRC's interactive rulemaking web site through the NRC Home Page (<http://www.nrc.gov>). This site provides the ability to upload comments as files (any format) if your web browser supports that function. For information about the interactive rulemaking web site, contact Ms. Carol Gallagher, (301) 415-5905; e-mail cag@nrc.gov. For information about Draft Regulatory Guide DG-1099, contact Mr. H.L. Graves at (301) 415-5880, e-mail hlg1@nrc.gov.

Although a time limit is given for comments on these draft guides, comments and suggestions in connection with items for inclusion in guides currently being developed or improvements in all published guides are encouraged at any time.

Regulatory guides are available for inspection at the NRC's Public Document Room, 11555 Rockville Pike, Rockville, MD; the PDR's mailing address is USNRC PDR, Washington, DC 20555; telephone (301) 415-4737 or (800) 397-4205; fax (301) 415-3548; e-mail pdr@nrc.gov. Requests for single copies of draft or final guides (which may be reproduced) or for placement on an automatic distribution list for single copies of future draft guides in specific divisions should be made in writing to the U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Reproduction and Distribution Services Section; or by e-