

337-5251, fax (610) 337-5269; or by email: kad@nrc.gov.

SUPPLEMENTARY INFORMATION:

I. Introduction

The U.S. Nuclear Regulatory Commission (NRC) is considering the issuance of a license amendment to Adolor Corporation for Materials License No. 37-30369-01, to authorize release of its facility in Malvern, Pennsylvania for unrestricted use and has prepared an Environmental Assessment (EA) in support of this action in accordance with the requirements of 10 CFR part 51. Based on the EA, the NRC has concluded that a Finding of No Significant Impact (FONSI) is appropriate.

II. EA Summary

The purpose of the proposed action is to allow for the release of the licensee's Malvern, Pennsylvania facility for unrestricted use. Adolor Corporation was authorized by NRC from August 20, 1997 to use radioactive materials for research and development purposes at the site. On July 18, 2003, Adolor Corporation requested that NRC release the facility for unrestricted use. Adolor Corporation has conducted surveys of the facility and determined that the facility meets the license termination criteria in subpart E of 10 CFR part 20.

III. Finding of No Significant Impact

The NRC staff has evaluated Adolor Corporation's request and the results of the surveys and has concluded that the completed action complies with 10 CFR part 20. The staff has prepared the EA (summarized above) in support of the proposed license amendment to terminate the license and release the facility for unrestricted use. On the basis of the EA, the NRC has concluded that the environmental impacts from the proposed action are expected to be insignificant and has determined not to prepare an environmental impact statement for the proposed action.

IV. Further Information

The EA and the documents related to this proposed action, including the application for the license amendment and supporting documentation, are available for inspection at NRC's Public Electronic Reading Room at <http://www.nrc.gov/reading-rm/adams.html> (ADAMS Accession Nos. ML031880271, ML032030484, ML032541074 and ML040060259). These documents are also available for inspection and copying for a fee at the Region I Office, 475 Allendale Road, King of Prussia, Pennsylvania, 19406.

Dated at King of Prussia, Pennsylvania this 7th day of January, 2004.

For the Nuclear Regulatory Commission.

John D. Kinneman,

*Chief, Nuclear Materials Safety Branch 2,
Division of Nuclear Materials Safety, Region I.*

[FR Doc. 04-787 Filed 1-13-04; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 72-17]

Portland General Electric Company, Trojan Independent Spent Fuel Storage Installation; Notice of Docketing of Materials License SNM-2509 Amendment Application

By letter dated December 9, 2003, Portland General Electric Company (PGE) submitted an application to the Nuclear Regulatory Commission (NRC or the Commission), in accordance with 10 CFR part 72, requesting the amendment of the Trojan Independent Spent Fuel Storage Installation (ISFSI) license (SNM-2509) and the Technical Specifications for the ISFSI located at Columbia County, Oregon. PGE is seeking Commission approval to amend the materials license and the ISFSI Technical Specifications to reflect completion of dry storage cask loading operations and incorporation of an administrative change to the Trojan ISFSI Technical Specifications to conform to a recent NRC Final Rule, "Event Notification Requirements," (63 FR 33611 dated June 5, 2003).

This application was docketed under 10 CFR part 72; the ISFSI Docket No. is 72-17 and will remain the same for this action. The amendment of an ISFSI license is subject to the Commission's approval.

The Commission may issue either a notice of hearing or a notice of proposed action and opportunity for hearing in accordance with 10 CFR 72.46(b)(1) or, if a determination is made that the amendment does not present a genuine issue as to whether public health and safety will be significantly affected, take immediate action on the amendment in accordance with 10 CFR 72.46(b)(2) and provide notice of the action taken and an opportunity for interested persons to request a hearing on whether the action should be rescinded or modified.

For further details with respect to this amendment, see the application dated December 9, 2003, which is publically available in the records component of NRC's Agencywide Documents Access and Management System (ADAMS). The NRC maintains ADAMS, which

provides text and image files of NRC's public documents. These documents may be accessed through the NRC's Public Electronic Reading Room on the Internet at <http://www.nrc.gov/reading-rm/adams.html>. If you do not have access to ADAMS or if there are problems in accessing the documents located in ADAMS, contact the NRC Public Document Room (PDR) Reference staff at 1-800-397-4209, 301-415-4737 or by email to pdr@nrc.gov.

Dated at Rockville, Maryland, this 31st day of December, 2003.

For the Nuclear Regulatory Commission.

Christopher M. Regan,

*Project Manager, Spent Fuel Project Office,
Office of Nuclear Material Safety and Safeguards.*

[FR Doc. 04-790 Filed 1-13-04; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50-263]

Nuclear Management Company, LLC, Monticello Nuclear Generating Plant Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of an amendment to Facility Operating License No. DPR-22, issued to Nuclear Management Company, LLC (NMC), for operation of the Monticello Nuclear Generating Plant (Monticello), located in Wright County, Minnesota. Therefore, as required by 10 CFR 51.21, the NRC is issuing this environmental assessment and finding of no significant impact.

Environmental Assessment

Identification of the Proposed Action

The proposed action would revise the Monticello operating license to change the Monticello design bases and the Updated Safety Analysis Report (USAR). The proposed action would revise the existing analyses for the following:

- Long-term containment response to the design-basis loss-of-coolant accident (LOCA).
- Containment overpressure (the pressure above the initial containment pressure) required for adequate available net positive suction head (NPSH) for the low-pressure emergency core cooling system (ECCS) pumps following a LOCA.

NMC intends to use these analyses to justify restoring the service water temperature to its licensing-basis value of 90 degrees F. NMC administratively

limits the service water temperature to 85 degrees F because the results of previous analyses of a scenario (reactor vessel isolation with high-pressure coolant injection being unavailable) showed that the design temperature for the piping attached to the wetwell would be exceeded. NMC's revised analyses shows the design temperature is not exceeded.

The proposed action is in accordance with NMC's application of December 6, 2002, as supplemented September 24, 2003.

The Need for the Proposed Action

NMC needs this license amendment because it has determined, in accordance with 10 CFR 50.59(c)(2)(viii), that the updated containment analyses involve different evaluation methods from those currently described in Monticello's USAR and previously approved by the NRC.

Environmental Impacts of the Proposed Action

The NRC staff reviewed NMC's amendment request and will issue a safety evaluation documenting its review. The NRC staff has reviewed NMC's calculation of the mass and energy releases that are used to determine containment pressure response, including the methods and key underlying input assumptions (e.g., decay heat generation).

NMC used conservative assumptions in its reanalyses which underestimate the containment pressure and overestimate the suppression pool water temperature. Some overpressure is necessary to ensure sufficient available NPSH. The conservative assumptions used in NMC's calculations and the cautions in Monticello's emergency operating procedures are intended to ensure that this pressure will be available.

The NRC has completed its evaluation of the proposed action and concludes, as set forth below, that there are no significant environmental impacts associated with the proposed changes to the Monticello design basis and USAR. The details of the NRC staff's review of the amendment request will be provided in the related safety evaluation when it is issued by the NRC.

The proposed action will not significantly increase the probability or consequences of accidents, no changes are being made in the types or amounts of effluents that may be released off site, and there is no significant increase in occupational or public radiation exposure. Therefore, there are no significant radiological environmental

impacts associated with the proposed action.

With regard to potential nonradiological impacts, the proposed action does not have a potential to affect any historic sites. It does not affect nonradiological plant effluents and has no other environmental impact. Therefore, there are no significant nonradiological environmental impacts associated with the proposed action.

Accordingly, the NRC concludes that there are no significant environmental impacts associated with the proposed action.

Environmental Impacts of the Alternatives to the Proposed Action

As an alternative to the proposed action, the staff considered denial of the proposed action (i.e., the "no-action" alternative). Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

The action does not involve the use of any different resource than those previously considered in the Final Environmental Statement for Monticello dated November 1972.

Agencies and Persons Consulted

On January 6, 2004, the staff consulted with the Minnesota State official, Nancy Campbell of the Department of Commerce, regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

On the basis of the environmental assessment, the NRC concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the NRC has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see NMC's letter of December 6, 2002, as supplemented September 24, 2003. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room (PDR), located at One White Flint North, Public File Area O1 F21, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the Agencywide Documents Access and Management System (ADAMS) Public Electronic Reading Room on the Internet at the NRC Web site, <http://www.nrc.gov/reading-rm/adams.html>.

Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS, should contact the NRC PDR Reference staff by telephone at 1-800-397-4209 or 301-415-4737, or by e-mail to pdr@nrc.gov.

Dated at Rockville, Maryland, this 6th day of January 2004.

For the Nuclear Regulatory Commission.

L. Raghavan,

Chief, Section 1, Project Directorate III, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 04-789 Filed 1-13-04; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards: Joint Meeting of the ACRS Subcommittees on Materials and Metallurgy and on Thermal-Hydraulic Phenomena; Notice of Meeting

The ACRS Subcommittees on Materials and Metallurgy and on Thermal-Hydraulic Phenomena will hold a joint meeting on February 3-4, 2004, Room T-2B3, 11545 Rockville Pike, Rockville, Maryland.

Portions of the meeting may be closed to public attendance to discuss Argonne National Laboratory (ANL) proprietary information per 5 U.S.C. 552b(c)(4).

The agenda for the subject meeting shall be as follows:

Tuesday and Wednesday, February 3-4, 2004—8:30 a.m. Until the Conclusion of Business

The Subcommittees will review the resolution of certain items identified by the ACRS in NUREG-1740, "Voltage-Based Alternative Repair Criteria," related to the Differing Professional Opinion on steam generator tube integrity, as well as the status of resolution of remaining items. The purpose of this meeting is to gather information, analyze relevant issues and facts, and formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Members of the public desiring to provide oral statements and/or written comments should notify the Designated Federal Official, Mr. Bhagwat P. Jain (telephone: 301-415-7270), five days prior to the meeting, if possible, so that appropriate arrangements can be made. Electronic recordings will be permitted.

Further information regarding this meeting can be obtained by contacting the Designated Federal Official between 7:30 a.m. and 5 p.m. (ET). Persons planning to attend this meeting are