Copyright Office. Affidavits received in this manner will be accepted with the understanding that the owners of those stations will resubmit affidavits when the Office next formally updates the specialty station list. An interested party may file an objection to any late–filed affidavit. Such objections shall be kept on file in the Copyright Office together with the corresponding affidavit.

February 2, 2007

Marybeth Peters,

Register of Copyrights.

[FR Doc. E7–2104 Filed 2–7–06; 8:45 am]

BILLING CODE 1410–30–S

NATIONAL CREDIT UNION ADMINISTRATION

Agency Information Collection Activities: Submission to OMB for Extension of a Currently Approved Collection; Comment Request

AGENCY: National Credit Union Administration (NCUA).

ACTION: Request for comment.

SUMMARY: The NCUA intends to submit the following information collection to the Office of Management and Budget (OMB) for review and clearance under the Paperwork Reduction Act of 1995 (Pub. L. 104–13, 44 U.S.C. Chapter 35). This information collection is published to obtain comments from the public.

DATES: Comments will be accepted until April 9, 2007.

ADDRESSES: Interested parties are invited to submit written comments to the NCUA Clearance Officer listed below:

Clearance Officer: Mr. Neil McNamara, National Credit Union Administration, 1775 Duke Street, Alexandria, VA 22314–3428, Fax No. 703–837–2861, E-mail: *mcnamara@ncua.gov.*

FOR FURTHER INFORMATION CONTACT: Requests for additional information or a copy of the information collection request, should be directed to Tracy Sumpter at the National Credit Union Administration, 1775 Duke Street, Alexandria, VA 22314–3428, or at (703) 518–6444.

SUPPLEMENTARY INFORMATION: Proposal for the following collection of information:

Title: Corporate Credit Union Monthly Call Report.

OMB Number: 3133–0067.

Form Number: NCUA 5310.

Type of Review: Recordkeeping, reporting and monthly.

Description: NCUA utilizes the information to monitor financial conditions in corporate credit unions, and to allocate supervision and examination resources.

Respondents: Corporate credit unions, or ''banker's banks'' for natural person credit unions.

Estimated No. of Respondents/Record keepers: 30.

Estimated Burden Hours per Response: 2 hours.

Frequency of Response: Monthly.

Estimated Total Annual Burden Hours: 720 hours.

Estimated Total Annual Cost: None.

By the National Credit Union Administration Board on February 5, 2007.

Mary Rupp,

Secretary of the Board.

[FR Doc. E7–2096 Filed 2–7–07; 8:45 am]

BILLING CODE 7535–01–P

NUCLEAR REGULATORY COMMISSION

Final Regulatory Guide: Issuance, Availability

The U.S. Nuclear Regulatory Commission (NRC) has issued a revision to an existing guide in the agency's Regulatory Guide Series. This series has been developed to describe and make available to the public such information as methods that are acceptable to the NRC staff for implementing specific parts of the NRC's regulations, techniques that the staff uses in evaluating specific problems or postulated accidents, and data that the staff needs in its review of applications for permits and licenses.

Like its predecessor, Revision 1 of Regulatory Guide 1.196, ''Control Room Habitability at Light-Water Nuclear Power Reactors,'' provides guidance and criteria that the staff of the U.S. Nuclear Regulatory Commission (NRC) considers acceptable for implementing the agency's regulations in Appendix A, ''General Design Criteria for Nuclear Power Plants,'' to Title 10, Part 50, of the *Code of Federal Regulations* (10 CFR part 50), ''Domestic Licensing of Production and Utilization Facilities,'' as they relate to control room habitability (CRH). Specifically, this guide outlines a process that licensees may apply to control rooms that are modified, are newly designed, or must have their conformance to the regulations reconfirmed.

In Appendix A to 10 CFR Part 50, General Design Criteria (GDC) 1, 3, 4, 5, and 19 apply to CRH, as follows:

- GDC 1, ''Quality Standards and Records,'' requires that structures, systems, and components (SSCs) important to safety be designed, fabricated, erected, and tested to quality standards commensurate with the importance of the safety functions performed.
- GDC 3, ''Fire Protection,'' requires that SSCs important to safety be designed and located to minimize the effects of fires and explosions.
- GDC 4, ''Environmental and Dynamic Effects Design Bases,'' requires SSCs important to safety to be designed to accommodate the effects of, and to be compatible with, the environmental conditions associated with normal operation, maintenance, testing, and postulated accidents, including loss-of-coolant accidents (LOCAs).
- GDC 5, ''Sharing of Structures, Systems, and Components,'' requires that SSCs important to safety not be shared among nuclear power units unless it can be shown that such sharing will not significantly impair their ability to perform their safety functions, including, in the event of an accident in one unit, the orderly shutdown and cooldown of the remaining units.
- GDC 19, ''Control Room,'' requires that a control room be provided from which actions can be taken to operate the nuclear reactor safely under normal conditions and to maintain the reactor in a safe condition under accident conditions, including a LOCA. Adequate radiation protection is to be provided to permit access and occupancy of the control room under accident conditions without personnel receiving radiation exposures in excess of specified values.

Since the NRC initially issued Regulatory Guide 1.196 in May 2003, the staff determined that the information presented in Appendix B to that guide did not accurately represent a viable technical specification for CRH at light-water nuclear power reactors. In particular, it referred to failure of a particular surveillance as a plant state, rather than having the results of the surveillance factor into the operability determination. In addition, it did not provide for a definite time to restore functionality to the control room envelope, whereas all improved standard technical specifications (iSTS) contain such provisions. Moreover, Appendix B was included as a ''strawman,'' to be deleted when details had been more carefully worked out with industry participation, and those technical specifications placed in the iSTS with all other acceptable technical specifications.

As of the publication date of this Revision 1 of Regulatory Guide 1.196, no utility has been granted the technical specification changes represented by

Appendix B to the original version of this guide. Consequently, the NRC staff elected to remove Appendix B (and all related references) from this revision. Removal of Appendix B from this revised guide does not require any stakeholder to take any action and does not reduce safety in any way. Moreover, public meetings with the owners' group Technical Specification Task Force have provided ample opportunity for public comment regarding this revision. Therefore, the staff views the removal of Appendix B as a neutral action, for which further public comments are unnecessary. For that reason, the staff chose not to issue this revision as a draft guide for public comment before publishing this Revision 1 of Regulatory Guide 1.196. Nonetheless, the NRC staff encourages and welcomes comments and suggestions in connection with improvements to published regulatory guides, as well as items for inclusion in regulatory guides that are currently being developed. You may submit comments by any of the following methods.

Mail comments to: Rulemaking, Directives and Editing Branch, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001.

Hand-deliver comments to: Rulemaking, Directives and Editing Branch, Office of Administration, U.S. Nuclear Regulatory Commission, 11555 Rockville Pike, Rockville, Maryland 20852, between 7:30 a.m. and 4:15 p.m. on Federal workdays.

Fax comments to: Rulemaking, Directives and Editing Branch, Office of Administration, U.S. Nuclear Regulatory Commission at (301) 415–5144.

Requests for technical information about Revision 1 of Regulatory Guide 1.196 may be directed to Harold Walker, at (301) 415–2827 or *HXW@nrc.gov.*

Regulatory guides are available for inspection or downloading through the NRC's public Web site at *http://www.nrc.gov/reading-rm/doc-collections/reg-guides/.* In addition, Revision 1 of Regulatory Guide 1.196 is available for inspection or downloading through ADAMS at *http://www.nrc.gov/reading-rm/adams.html,* under Accession #ML063560144.

Revision 1 of Regulatory Guide 1.196 and other related publicly available documents can also be viewed electronically on computers in the NRC's Public Document Room (PDR), which is located at 11555 Rockville Pike, Rockville, Maryland. The PDR's reproduction contractor will make copies of documents for a fee. The PDR's mailing address is USNRC PDR, Washington, DC 20555–0001. The PDR can also be reached by telephone at (301) 415–4737 or (800) 397–4205, by fax at (301) 415–3548, and by e-mail to *PDR@nrc.gov.*

Please note that the NRC does not intend to distribute printed copies of Revision 1 of Regulatory Guide 1.196, unless specifically requested on an individual basis with adequate justification. Such requests for single copies of draft or final guides (which may be reproduced) should be made in writing to the U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, Attention: Reproduction and Distribution Services Section; by e-mail to *DISTRIBUTION@nrc.gov;* or by fax to (301) 415–2289. Telephone requests cannot be accommodated.

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(5 U.S.C. 552(a))

Dated at Rockville, Maryland, this 23rd day of January, 2007.

For the U.S. Nuclear Regulatory Commission.

Brian W. Sheron,

Director, Office of Nuclear Regulatory Research.

[FR Doc. E7–2088 Filed 2–7–07; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

Final Regulatory Guide: Issuance, Availability

The U.S. Nuclear Regulatory Commission (NRC) has issued a revision to an existing guide in the agency's Regulatory Guide Series. This series has been developed to describe and make available to the public such information as methods that are acceptable to the NRC staff for implementing specific parts of the NRC's regulations, techniques that the staff uses in evaluating specific problems or postulated accidents, and data that the staff needs in its review of applications for permits and licenses.

Revision 1 of Regulatory Guide 1.200, ''An Approach for Determining the Technical Adequacy of Probabilistic Risk Assessment Results for Risk-Informed Activities,'' describes one acceptable approach for determining whether the quality of a probabilistic risk assessment (PRA), in total or the parts that are used to support an application, is sufficient to provide confidence in the results, such that the PRA can be used in regulatory decision-making for light-water reactors. Specifically, Revision 1 of Regulatory Guide 1.200 provides guidance in four areas:

(1) A minimal set of requirements of a technically acceptable PRA.

(2) The NRC's position on PRA consensus standards and industry PRA program documents.

(3) Demonstration that the PRA (in total or specific parts) used in regulatory applications is of sufficient technical adequacy.

(4) Documentation to support a regulatory submittal.

This guidance is intended to be consistent with the NRC's PRA Policy Statement, entitled ''Use of Probabilistic Risk Assessment Methods in Nuclear Activities: Final Policy Statement,'' which the NRC published in the **Federal Register** on August 16, 1995 (60 FR 42622) to encourage use of PRA in all regulatory matters. That Policy Statement states that ''* * * the use of PRA technology should be increased to the extent supported by the state-of-the-art in PRA methods and data and in a manner that complements the NRC's deterministic approach.''

Since that time, many uses have been implemented or undertaken, including modification of the NRC's reactor safety inspection program and initiation of work to modify reactor safety regulations. Consequently, confidence in the information derived from a PRA is an important issue, in that the accuracy of the technical content must be sufficient to justify the specific results and insights that are used to support the decision under consideration.

Revision 1 of Regulatory Guide 1.200 is also intended to be consistent with the more detailed guidance in Regulatory Guide 1.174, ''An Approach for Using Probabilistic Risk Assessment in Risk-Informed Decisions on Plant-Specific Changes to the Licensing Basis,'' which the NRC issued in November 2002. In addition, Revision 1 of Regulatory Guide 1.200 is intended to reflect and endorse (with certain objections) the following guidance provided by the American Society of Mechanical Engineers (ASME) and the Nuclear Energy Institute (NEI):

- ASME RA–S–2002, ''Standard for Probabilistic Risk Assessment for Nuclear Power Plant Applications,'' dated April 5, 2002.
- ASME RA–Sa7–2003, ''Standard for Probabilistic Risk Assessment for Nuclear Power Plant Applications,'' Addendum A to ASME RA–S–2002, dated December 5, 2003.
- ASME RA–Sb–2005, ''Standard for Probabilistic Risk Assessment for Nuclear Power Plant Applications,''