

Activity for Which Permit Is Requested

Waste Permit; The Earth Vision Trust proposes to install a maximum of 10 cameras distributed between 5 sites that are often visited by tourists. No more than 2 cameras would be installed at any one site. Cameras would be placed in such a way as to not disrupt wildlife. Cameras would be secured using 6–8 rock bolts drilled into rock outcrops. Each camera would be powered by a 10w solar panel and a sealed 12 volt 55 AH gel battery. The batteries would be housed in a leak proof plastic case. The cameras would remain deployed for 5 years and would be completely removed (including bolts and power sources) at the conclusion of the project. Each camera would be visited every 1–2 years to retrieve data, make necessary repairs, and remove non-functioning equipment. The cameras would be used to measure ice velocity and monitor the calving front of numerous outlet glaciers. The data would help advance scientific knowledge on the mechanics and pace of glacial retreat. Images gained from the cameras would also be used in global outreach campaigns to educate the public about the speed of climate change's impact on the earth.

Location

Five visitor sites in the Western Antarctic Peninsula Region: Paulet Island, Cierva Cove, Neko Harbor, Wiggins Glacier, and Gunnel Channel.

Dates

February 1, 2014 to March 31, 2019.

Nadene G. Kennedy,
Division of Polar Programs.

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NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50–458; NRC–2013–0190]

Entergy Operations, Inc., River Bend Station, Unit 1

AGENCY: Nuclear Regulatory Commission.

ACTION: Environmental assessment and finding of no significant impact; issuance.

ADDRESSES: Please refer to Docket ID NRC–2013–0190 when contacting the NRC about the availability of information regarding this document. You may access publicly-available information related to this action by the following methods:

- *Federal Rulemaking Web site:* Go to <http://www.regulations.gov> and search

for Docket ID NRC–2013–0190. Address questions about NRC dockets to Carol Gallagher; telephone: 301–287–3422; email: Carol.Gallagher@nrc.gov. For technical questions, contact the individual(s) listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- *NRC's Agencywide Documents Access and Management System (ADAMS):* You may access publicly available documents online in the NRC Library at <http://www.nrc.gov/reading-rm/adams.html>. To begin the search, select “ADAMS Public Documents” and then select “Begin Web-based ADAMS Search.” For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1–800–397–4209, 301–415–4737, or by email to pdr.resource@nrc.gov. The ADAMS accession number for each document referenced in this notice is provided the first time that a document is referenced. The application for exemption dated August 23, 2012, is available in ADAMS under Accession No. ML12241A250.

- *NRC's PDR:* You may examine and purchase copies of public documents at the NRC's PDR, Room O1–F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

SUPPLEMENTARY INFORMATION:

I. Introduction

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of an exemption from part 50 of Title 10 of the *Code of Federal Regulations* (10 CFR), appendix J for Facility Operating License No. NPF–47, issued to Entergy Operations, Inc. (the licensee), for operation of the River Bend Station, Unit 1 (RBS), located in West Feliciana Parish, Louisiana. Therefore, as required by 10 CFR 51.21, the NRC performed an environmental assessment. Based on the results of the environmental assessment, the NRC is issuing a finding of no significant impact.

II. Environmental Assessment

Identification of the Proposed Action

The proposed action would exempt the licensee from certain provisions of 10 CFR part 50, appendix J, “Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors.” This appendix requires that components which penetrate containment be periodically leak tested at the “ P_a ,” defined as the “calculated peak containment internal pressure related to the design basis accident specified either in the technical specification or associated bases.” The NRC noted a

conflict between Entergy's interpretation of P_a and the literal reading of the definition of P_a in the regulations.

For the extended power uprate, Entergy had re-performed the containment pressure analysis and determined that the calculated peak pressure in containment occurs in a localized area of the wetwell within a few seconds after a postulated main steamline break. The NRC believes that as defined in the regulations the value of P_a should have been revised. The new calculation demonstrates that the localized pressure in the wetwell quickly drops and equalizes throughout the containment to a value of 3.6 pounds per square inch gauge (psig). Entergy has stated it believes the new calculated long-term peak containment pressure of 3.6 psig is the correct value to be used for P_a . However, to avoid a large number of procedural changes to reflect this new peak value, Entergy did not propose to change the current Technical Specification (TS) value of P_a (7.6 psig).

The exemption would allow RBS to continue to use the pre-extended power uprate value of 7.6 psig rather than use the newly calculated localized pressure spike value of 9.3 psig in the wetwell for P_a . The NRC staff examined the licensee's rationale to support the exemption request and concluded that the use the value of 7.6 psig for P_a would meet the underlying purpose of 10 CFR part 50, appendix J. Supporting the use of this alternate value is:

(1) The time for the pressure spike to occur and fall to equilibrium is 6 seconds, which is not sufficient time to release source terms from the core,

(2) the pressure spike is also localized to the wetwell area which makes up roughly 10 percent of containment,

(3) the number of containment penetrations in this area is limited. Therefore, the current P_a value of 7.6 psig meets the intent of 10 CFR part 50, appendix J by bounding the peak bulk containment pressure (3.6 psig) and assuring that leakage through the primary containment does not exceed allowable leakage rate values,

(4) the calculated peak bulk containment pressure is 3.6 psig so the TS value of 7.6 is conservative for the use of determining containment leakage, and

(5) this request is consistent with the determination that the NRC staff has reached for other licensees under similar conditions based on the same considerations.

Therefore, the P_a TS value of 7.6 psig meets the intent of 10 CFR part 50, appendix J by bounding the peak bulk

containment pressure and assuring that leakage through the primary containment does not exceed allowable leakage rate values.

The proposed exemption would allow RBS to continue to use an alternate definition of P_a of 7.6 psig. This use of the alternate definition for P_a meets the intent of 10 CFR part 50, appendix J because it provides testing of the primary containment parameters at a bounding pressure that is calculated to be possible throughout containment over a sustained period following a design basis accident.

The Need for the Proposed Action

The proposed exemption is needed to allow RBS to continue to use an alternate definition for P_a which results in the use of a lower pressure for appendix J containment testing. Use of the lower pressure reduces the burden of modifying the test procedures, seeking NRC authorization to change the current TS value, and conducting the testing at the higher pressure. In addition, applying the literal definition for P_a would not serve the underlying purpose of the rule which is to test the primary containment parameters at a peak pressure calculated to exist over the long term following a design basis accident.

Environmental Impacts of the Proposed Action

The NRC has completed its evaluation of the proposed action and concludes that the exemption described above meets the intended purpose of the requirements in 10 CFR part 50, appendix J. The lower calculated P_a value provides a representative bounding pressure for evaluating the leak-tight integrity of the primary reactor containment and related penetrations.

The proposed action will not significantly increase the probability or consequences of accidents. No changes are being made in the types of effluents that may be released offsite. There is no significant increase in the amount of any effluent released offsite. There is no significant increase in occupational or public radiation exposure. Therefore, there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential non-radiological impacts, the proposed action does not have any foreseeable impacts to land, air, or water resources, including impacts to biota. In addition, there are also no known socioeconomic or environmental justice impacts associated with such proposed action. Therefore, there are no significant non-

radiological environmental impacts associated with the proposed action.

Accordingly, the NRC concludes that there are no significant environmental impacts associated with the proposed action.

Environmental Impacts of the Alternatives to the Proposed Action

As an alternative to the proposed action, the staff considered denial of the proposed action (i.e., the “no-action” alternative). Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

The action does not involve the use of any different resources than those previously considered in the “Final Environmental Statement,” NUREG-1073, January 1985, for the RBS.

Agencies and Persons Notified

In accordance with its stated policy, on August 6, 2013, the staff notified the Louisiana State official, Ji Wiley, of the Louisiana Department of Environmental Quality, Radiation Protection Division, regarding the environmental impact of the proposed action. The State official had no comments.

III. Finding of No Significant Impact

On the basis of the environmental assessment, the NRC concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the NRC has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated August 23, 2012.

Dated at Rockville, Maryland, this 9th day of August 2013. For the Nuclear Regulatory Commission.

Michael T. Markley,

Chief, Plant Licensing Branch IV, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

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NUCLEAR REGULATORY COMMISSION

[Docket Nos. 52-025, and 52-026; NRC-2008-0252]

Vogtle Electric Generating Station, Units 3 and 4; Southern Nuclear Operating Company; Changes to the Chemical Volume Control System

AGENCY: Nuclear Regulatory Commission.

ACTION: Exemption and combined license amendment; issuance.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is both granting an exemption to allow a departure from the certification information of Tier 1 of the generic design control document (DCD) and issuing License Amendment No. 12 to Combined Licenses (COL), NPF-91 and NPF-92. The COLs were issued to Southern Nuclear Operating Company, Inc., and Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and the City of Dalton, Georgia (the licensee), for construction and operation of the Vogtle Electric Generating Plant (VEGP), Units 3 and 4, located in Burke County, Georgia. The amendment requests changes that modify the Chemical and Volume Control System (CVS), including changes to information located in Tier 1 Tables 2.3.2-1 and 2.3.2-2, and Tier 1 Figures 2.2.1-1 and 2.3.2-1. The granting of the exemption allows the changes to Tier 1 information as specified in the license amendment request. Because the acceptability of the exemption was determined in part by the acceptability of the amendment, the exemption and amendment are being issued concurrently.

ADDRESSES: Please refer to Docket ID NRC-2008-0252 when contacting the NRC about the availability of information regarding this document. You may access information related to this document, which the NRC possesses and is publicly available, using any of the following methods:

- Federal Rulemaking Web site: Go to <http://www.regulations.gov> and search for Docket ID NRC-2008-0252. Address questions about NRC dockets to Carol Gallagher; telephone: 301-287-3422; email: Carol.Gallagher@nrc.gov. For technical questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- NRC's Agencywide Documents Access and Management System (ADAMS): You may access publicly available documents online in the NRC Library at <http://www.nrc.gov/reading-rm/adams.html>. To begin the search,